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Radiological Protection in Therapy with Radiopharmaceuticals

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DRAFT REPORT FOR CONSULTATION: DO NOT REFERENCE

EDITORIAL

To be drafted

ABSTRACT

Radiological Protection in Therapy with Radiopharmaceuticals

ICRP Publication 1XX

Approved by the Commission in ____ 201X

Abstract-The use of radiopharmaceuticals for therapy using novel radionuclides, compounds, tracer molecules, and the administration techniques is increasing for the treatment of various tumours. The goal of radiation therapy, including therapy with radiopharmaceuticals, is to optimise the relationship between the probability of control of tumour/target tissue and complications in normal tissue. Essential to this optimisation is ability to quantify radiation dose to both tumour/target tissue and normal tissue. This report provides a framework for calculating radiation doses for various treatment approaches. In radiopharmaceutical therapy, the absorbed dose in an organ or tissue is governed by the radiopharmaceutical uptake, retention in and clearance from the various organs and tissues of the body, together with radionuclide physical half-life. These biokinetic data are based on measurements made using techniques that vary in complexity and the required accuracy will depend on the specific application. For treatment planning, absorbed dose calculations are performed prior to therapy using a trace-labelled diagnostic administration, or post-therapy on the basis of the therapy administration. Uncertainty analyses provide additional information about sources of bias and random variation and their magnitudes; these analyses show the reliability and quality of absorbed dose calculations. Effective dose can provide a measure of lifetime risk of detriment attributable to the stochastic effects of radiation exposure, principally cancer, but effective dose does not apply to short-term deterministic effects associated with radiopharmaceutical therapy. Accident prevention in radiation therapy should be an integral part of the design of facilities, equipment, and administration procedures. Optimisation of staff exposures includes consideration of equipment design, proper shielding and handling of sources, and personal protective equipment and tools, as well as education and training to promote awareness and engagement in radiation protection. The decision to hold or release a patient after radiopharmaceutical therapy should take account of estimates of possible radiation dose to members of the general public and carers from residual activity in the patient. In these situations, specific radiation protection guidance should be provided to patients and caregivers.

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Keywords: Radiopharmaceutical therapy; Radionuclide; Dose estimation; Radiological protection

AUTHORS ON BEHALF OF ICRP

PREFACE

Over the years, the International Commission on Radiological Protection (ICRP), referred below as ‘the Commission’, has issued many reports providing advice on radiological protection and safety in medicine. *Publication 105* is a general overview of this area (ICRP, 2007b). These reports summarise the general principles of radiological protection, and provide advice on the application of these principles to the various uses of ionising radiation in medicine.

The use of radiopharmaceuticals for therapy is increasing for the treatment of various tumours using novel radionuclides, compounds, tracer molecules, and the administration techniques. Radiopharmaceutical therapy is of benefit for the patient; optimising patient benefit implies optimising the factors that are most likely to contribute to positive responses to therapy. The medical community currently does not have easy access to methods and protocols for the collection of useful biokinetic or dosimetric data on such approaches. The report is intended to provide information on reasonable and practical approaches for the management of patient dose in therapy with radiopharmaceuticals as well as for protection of staff and members of the public.

Although ICRP published various recommendations for the use of radiopharmaceuticals, there have been no reports specific to radiopharmaceutical therapy. At the meeting in Bethesda, 2011, the Committee 3 discussed the need for a new report and proposed to establish a working party. The Commission launched a Task Group on Radiological Protection in Therapy with Radiopharmaceuticals in 2016.

The membership of the Task Group 101 was as follows:

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1

MAIN POINTS

- 2 • Treatment with radiopharmaceuticals requires the development of administration
3 protocols that justify and optimise the treatment. Individual absorbed dose
4 estimates should be performed for treatment planning and post-administration
5 verification of doses received by tumour and normal tissues, as radiation delivered
6 to normal tissues can cause tissue reactions and there is a risk of secondary
7 malignancies.
- 8 • Special consideration should be given to pregnant women (and children) exposed to
9 ionising radiation. Pregnancy is a strong contraindication to radiopharmaceutical
10 therapy, unless the therapy is life-saving. Breastfeeding should be discontinued in
11 radiopharmaceutical therapy patients.
- 12 • Radiation sources used in radiopharmaceutical therapy can contribute significant
13 doses to medical personnel and others who may spend time within or adjacent to
14 rooms that contain such sources. Meaningful dose reduction and contamination
15 control can be achieved through the use of appropriate procedures, and facility and
16 room design, including shielding where appropriate, as well as education and
17 training to promote awareness and engagement in radiation protection. Accident
18 prevention and review of near misses in radiopharmaceutical therapy should be an
19 integral part of the design of facilities, equipment, and administration procedures.
- 20 • Medical practitioners should provide all necessary medical care consistent with
21 patient safety and appropriate medical management. Radiation protection
22 considerations should not prevent or delay life-saving medical procedures or
23 surgery in the event that they may be required/helpful. Staff should be informed
24 when a patient may pose a radioactive hazard, and advice and training should be
25 provided.
- 26 • The decision to hospitalise or release a patient after therapy should be made based
27 on existing guidance and regulations, as well as on the individual patient situation,
28 considering factors such as the residual activity in the patient, the patient's wishes,
29 family considerations (particularly the presence of children or pregnant family
30 members), and environmental factors. Information on specific radiation protection
31 precautions should be provided to patients and carers.
- 32

33

34

GLOSSARY

35

36 Absorbed dose, *D*

37 The quotient of the mean energy (*d*) imparted to an element of matter by ionising
38 radiation and the mass (*dm*) of the element.

39
$$D = \frac{d\bar{e}}{dm}$$

40 Absorbed dose is the basic physical dose quantity and is applicable to all types of
41 ionising radiation and to any material. Absorbed dose is a measurable quantity for
42 which primary standards exist. In the International System of Units (SI), the unit for
43 absorbed dose is joule per kilogramme (J kg⁻¹), and its special name is gray (Gy).

44 Ambient dose equivalent, *H*(10)*

45 The dose equivalent at a point in radiation field that would be produced by the
46 corresponding expanded and aligned field in the ICRU sphere at depth of 10 mm on
47 the radius opposing the direction of the aligned field. The unit of ambient dose
48 equivalent is joule per kilogram (J kg⁻¹) and its special name is sievert (Sv).

49 Biologically effective dose (BED)

50 A concept within the linear-quadratic cell survival model, used to calculate the
51 different absorbed doses required to produce the same probability of a specified
52 biological endpoint, when the absorbed doses are delivered with different
53 fractionation schemes or absorbed-dose rate patterns. Theoretically, the BED is the
54 absorbed dose that would be required to produce a specified biological endpoint, if
55 the dose were delivered by infinitesimally small dose fractions, or at a very low dose
56 rate.

57 Comforters and carers

58 Individuals, other than staff, who care for and comfort patients. These individuals
59 include parents and others, normally family or close friends who hold children during
60 diagnostic procedures or may close to patients following the administration of
61 radiopharmaceuticals or during brachytherapy (ICRP, 2007a).

62 Deterministic effect

63 Injury in populations of cells, characterised by a threshold dose and an increase in the
64 severity of the reaction as the dose is increased further. Deterministic effect is also
65 termed a ‘tissue reaction’. In some cases, deterministic effects are modifiable by post-
66 irradiation procedures including biological response modifiers (ICRP, 2007a).

67 Dose equivalent, *H*

68 The product of *D* and *Q* at a point in tissue, where *D* is the absorbed dose and *Q* is the
69 quality factor for the specific radiation at this point, thus:

70 $H = D \cdot Q$

71 The unit of dose equivalent is joule per kilogramme (J kg^{-1}), and its special name is
72 sievert (Sv).

73 Dose limit

74 The value of the effective dose received by an individual within a specified period
75 from planned exposure situations that shall not be exceeded. Dose limitation is one of
76 three fundamental principles of radiological protection originally defined by ICRP.

77 Effective dose, E

78 The tissue-weighted sum of the equivalent doses in all specified tissues and organs of
79 the body, given by the expression:

80
$$E = \sum_T w_T \sum_R w_R D_{T,R}$$

81 where $D_{T,R}$ is the mean absorbed dose from radiation R in a tissue or organ, T, and w_T
82 is the tissue weighting factor and w_R is the radiation weighting factor. The unit for the
83 effective dose is the same as for absorbed dose (J kg^{-1}), and its special name is sievert
84 (Sv).

85 Justification

86 One of three fundamental principles of radiological protection originally defined by
87 ICRP. The process of determining whether: (i) a planned activity involving radiation
88 is beneficial overall (i.e. whether the benefits to individuals and to society from
89 introducing or continuing the activity outweigh the harm resulting from the activity);
90 or (ii) the decision to control exposure in an emergency or existing exposure situation
91 is likely to be beneficial overall (i.e. whether the benefits to individuals and society
92 outweigh its cost and any harm or damage it causes).

93 Linear energy transfer (LET)

94 The average linear energy loss of charged particle radiation in a medium, i.e., the
95 radiation energy lost per unit length of path through a material. That is, the quotient of
96 dE by dl where dE is the mean energy lost by a charged particle owing to collisions
97 with electrons in traversing a distance dl in matter.

98
$$L = \frac{dE}{dl}$$

99 The unit of L is J m^{-1} , often given in $\text{keV } \mu\text{m}^{-1}$.

100 Occupational exposure

101 All exposure incurred by workers in the course of their work, with the exception of:
102 (1) excluded exposures and exposures from exempt activities involving radiation or
103 exempt sources; (2) any medical exposure; and (3) the normal local natural
104 background radiation.

105 Optimisation of protection

106 The principle of optimisation of radiological protection is a source-related process
 107 that aims to keep the magnitude of individual doses, the number of people exposed,
 108 and the likelihood of potential exposure as low as reasonably achievable below the
 109 appropriate dose criteria (constraint or reference level), economic and societal factors
 110 being taken into account.

111 Organ at risk (OAR)

112 Organs that might be damaged during exposure to radiation. It most frequently refers
 113 to healthy organs located in the radiation field during radiotherapy.

114 Quality factor, $Q(L)$

115 The factor characterising the biological effectiveness of a radiation, based on the
 116 ionisation density along the tracks of ion beams in tissue. Q is defined as a function of
 117 the unrestricted linear energy transfer, L_∞ (often denoted as L or LET), of ion beams
 118 in water:

119

$$Q(L) = \begin{cases} 1 & L < 10 \text{ keV}/\mu\text{m} \\ 0.32L - 2.2 & 10 \leq L \leq 100 \text{ keV}/\mu\text{m} \\ 300/\sqrt{L} & L > 100 \text{ keV}/\mu\text{m} \end{cases}$$

120

121 Q has been replaced by the radiation weighting factor, but it is still used in calculating
 122 the operational dose equivalent quantities used in monitoring.

123 Radiation detriment

124 A concept used to quantify the harmful health effects of radiation exposure in
 125 different parts of the body. It is defined by the Commission as a function of several
 126 factors, including incidence of radiation-related cancer or heritable effects, lethality of
 127 these conditions, quality of life, and years of life lost owing to these conditions.

128 Radiation induced second cancer

129 Ionising radiation has paradoxical aspects in both beneficial effects of curing cancer
 130 and the risk of inducing cancer. Induction of cancer by medium to high dose of
 131 radiation has been demonstrated by the significant increase in the incidence of
 132 cancers among workers handling radioactive substances and among atomic bomb
 133 survivors, as well as among survivors after radiotherapy.

134 Radiation weighting factor, w_R

135 A dimensionless factor by which the organ or tissue absorbed dose is weighted to
 136 reflect the higher biological effectiveness of high-LET radiations compared with low-
 137 LET radiations.

138 Relative biological effectiveness (RBE)

139 The ratio of absorbed dose of a low-LET reference radiation to absorbed dose of the
 140 radiation considered that gives an identical biological effect. RBE values vary with
 141 absorbed dose, dose rate, and biological endpoint considered.

142 Risk

143 Risk relates to the probability that an outcome (e.g. cancer) will occur. Terms relating
144 to risk are grouped together here:

145 Relative risk is the rate of disease in an exposed population divided by the rate of the
146 disease in an unexposed population.

147 Excess relative risk is the rate of disease in an exposed population divided by the rate
148 of the disease in an unexposed population minus 1. This is often expressed as the
149 excess relative risk per Sv.

150 Stochastic effect

151 The induction of malignant disease or heritable effects, for which the probability of an
152 effect occurring, but not its severity, is regarded for the purpose of radiological
153 protection to be increasing with the dose without a threshold.

154 Tissue weighting factor, w_T

155 The factor by which the equivalent dose to a tissue or organ T is weighted to
156 represent the relative contribution of that tissue or organ to the total health detriment
157 resulting from uniform irradiation of the body (ICRP, 2007b). It is weighted such
158 that:

159
$$\sum_T w_T = 1$$

160 Voxel phantom

161 Computational anthropomorphic phantom based on medical tomographic images
162 where the anatomy is described by small three-dimensional volume elements (voxels)
163 specifying the density and the atomic composition of the various organs and tissues of
164 the human body.

165

166

1. INTRODUCTION

167 (1) In radiation therapy, including therapy with radiopharmaceuticals, the dose to the
168 patient is intentional and its potentially cell-killing properties are the very purpose of the
169 treatment. In such cases, optimisation becomes an effort in minimising doses (and/or their
170 deleterious effects) to surrounding tissues without compromising the pre-determined and
171 intentionally lethal dose and effect on the target region. Basically, the aim is to eradicate the
172 neoplastic target tissue or to palliate the patient's symptoms. If the dose to the target tissue is
173 too low, the therapy will be ineffective and the exposure is not justified. The emphasis should
174 be on the justification of the medical procedures and on the optimisation of treatment and of
175 protection. Current ICRP recommendations related to therapy with radiopharmaceuticals are
176 found in ICRP *Publications* 73 (ICRP, 1996a), 94 (ICRP, 2004), 103 (ICRP, 2007a), 105
177 (ICRP, 2007b) and 128 (ICRP, 2015a).

178 (2) The medical community currently does not have easy access to methods and protocols
179 for the collection of useful biokinetic or dosimetric data for such procedures. Many centres,
180 even academic facilities, do not have such methods available despite performing research in
181 this area. This severely constrains development. As quantitative imaging and dosimetry is
182 seldom performed, many treatments are not appropriately optimised. Quantitative imaging
183 and dosimetry should be the basis for treatment planning for radiopharmaceutical therapy¹
184 just as it is for external beam radiotherapy.

185 (3) A collection and review of the existing information and literature in the context of
186 therapeutic uses will help to optimise therapeutic use of radiopharmaceuticals, particularly for
187 newer approaches. It is essential to alert the community to the variation in patient kinetics at
188 therapeutic levels of activity. This information can facilitate the introduction of new
189 radiopharmaceuticals, particularly with regard to the levels of the administered activity
190 prescribed.

191 (4) In general, there are many papers dealing with absorbed doses delivered to critical
192 organs and to tumours. Many of these include varying degrees of detail on the biokinetics of
193 uptake and retention, and uptake phases are often assumed to be instantaneous rather than
194 measured. The focus on therapy procedures has been on the absorbed doses delivered, so that
195 the biokinetics have not always been detailed. This information is presumably available. It
196 would be very valuable if biokinetic information for the increasing number of studies that are
197 being performed could be compiled and made publicly available. It would also be beneficial
198 to assess the integrity of data gathered from descriptions of the methods used to acquire the
199 data.

200 (5) The report is intended to explore, provide, and explain a framework for estimating
201 dosimetry for novel treatment approaches and identify those situations with unique aspects
202 that should be considered. Such a framework includes items such as: individual dosimetry to
203 plan the therapy, test activities and pre-treatment tracers, measurement of whole
204 body/tumour/organ kinetics, analysis of urine or blood samples, quantitative measurements of
205 the test activity; absorbed dose calculation based on 3D-patient images or patient-like

¹ Therapy with radiopharmaceutical is also referred to by many terms, including '(targeted) radionuclide therapy', 'unsealed source therapy', 'systemic radiation therapy' and 'molecular radiotherapy'. In this publication the generic term 'radiopharmaceutical therapy' is used to be consistent with other ICRP and ICRU publication.

206 phantoms using Monte-Carlo or analytical techniques, an evaluation of how to scale-up to
207 therapeutic activity levels, and written guidelines for the therapy.

208 (6) Dosimetry is necessary to provide justification for treatment with radiation with
209 respect to both the deterministic and stochastic effects. Radiopharmaceutical therapy practice
210 and optimisation require involvement of representatives of different competencies, including
211 medical physicists, nuclear medicine technologists, nuclear medicine physicians,
212 endocrinologists and oncologists.

213 (7) The target audience includes; nuclear medicine physicians and oncologists, medical
214 physicists, clinicians, practitioners and prescribers/referrers, radiopharmacists and nuclear
215 medicine technologists, radiation protection officers, regulatory authorities, medical and
216 scientific societies, industry, patients, patient advocacy groups and public protection officers.
217

218 **2. RADIOPHARMACEUTICAL THERAPY METHODS:**
219 **JUSTIFICATION AND OPTIMISATION**

- 220 • **Treatment with radiopharmaceuticals involves the development of administration**
221 **protocols that justify and optimise the treatment. Individual absorbed dose**
222 **estimates should be performed for treatment planning and post-administration**
223 **verification of doses received by tumour and normal tissues. Records of individual**
224 **dose estimates should be kept.**
- 225 • **Excess radiation delivered to normal tissues can cause tissue reactions and there are**
226 **risks of secondary malignancies. Dosimetry should be performed for each treatment,**
227 **particularly to children and young people.**
- 228 • **In ¹³¹I treatment of hyperthyroidism, a fully personalised approach based on**
229 **patient-specific measurements can ensure that the administered activity is the**
230 **minimum required for an effective treatment, thereby minimising the potential for**
231 **long term risks, offering the potential to render patients euthyroid, and also**
232 **minimising the radiation doses delivered to patients, staff, family and comforters**
233 **and carers.**
- 234 • **For ¹³¹I treatment of differentiated thyroid cancer, limited survival for high risk**
235 **patients indicates the need for stratification in treatment. To optimise treatments,**
236 **dosimetry should be performed for each treatment following therapy and further**
237 **studies are needed to investigate the role of pre-therapy dosimetry planning.**
- 238 • **Radiopharmaceuticals that target bone tissues, such as the beta particle emitters**
239 **⁸⁹Sr chloride and ¹⁵³Sm-EDTMP, have important roles in the management of**
240 **painful bone metastases, but optimal treatment protocols are not yet established.**
241 **An investigation of the optimal absorbed dose to deliver for the alpha emitter, ²²³Ra,**
242 **would help to determine optimal treatment regimens.**
- 243 • **In ¹³¹I-mIBG treatment of neuroblastoma in children and young adults, the**
244 **probability of inducing acute myelotoxicity, the potential for secondary neoplasms**
245 **and the need to justify administrations of high activity to children and young people**
246 **underline the need for personalised dosimetry planning and verification.**

247 **2.1. Introduction**

248 (8) Radiopharmaceutical therapy is a complex procedure, encompassing a wide range of
249 radionuclides, different targeting mechanisms and various methods of administration. Each
250 radiotherapeutic procedure presents a unique set of challenges for dosimetry calculations,
251 related either to quantitative imaging, the absorbed dose calculations themselves or
252 considerations of the deterministic or stochastic biological effects. The combined need for a
253 highly multidisciplinary approach and the relatively small number of patients treated, has
254 resulted in a lack of development within the field compared with that for external beam
255 radiotherapy (NCRP, 2006).

256 (9) Treatment objectives vary. In many cases the intention is to provide a palliative effect,
257 as in the case of beta emitters for bone metastases from castration resistant prostate cancer
258 (CRPC). In limited cases, as for the ablation of thyroid remnants following thyroidectomy,
259 complete responses are common. In the majority of treatments, a range of responses are seen.

260 (10) Radionuclide therapy using ^{131}I -iodide for the treatment of thyrotoxicosis and
261 thyroid cancer, and ^{32}P -orthophosphate for polycythaemia and for palliation of bone pain, has
262 been practised for over 70 years. The technique is increasingly being used for the treatment of
263 various tumours using several novel radionuclides, compounds, tracer molecules, and
264 application techniques. Examples of recently developed methods used in clinical practice are
265 ^{177}Lu -labelled peptides for therapy of neuroendocrine tumours and ^{223}Ra -dichloride for
266 treatment of painful bone metastases.

267 (11) It is important that the clinical introduction of a new radiotherapeutic method
268 involves the development of administration protocols that justify and optimise the treatment
269 and are not simply based on existing procedures for different radiopharmaceuticals
270 administered for different indications.

271 (12) At present, there are known to be a large number of radiotherapeutics in
272 development. Each new agent must be considered separately and the potential benefits and
273 risks involved must be considered in relation to individual patient status and the aim of
274 treatment.

275 (13) Records of the specifics of therapy with unsealed radionuclides should be
276 maintained at the hospital. Data from dose planning and about administered activity should
277 be included in the patients' records.

278 (14) Dose coefficients presented in ICRP *Publications 128* (ICRP, 2015a), *106* (ICRP,
279 2008) as well as *80* (ICRP, 1998) and *53* (ICRP, 1987) are intended for diagnostic nuclear
280 medicine and not for therapeutic applications. The use of radiopharmaceuticals for therapy
281 requires more detailed and patient-specific dosimetry and dose planning, including both
282 tumour and normal tissue.

283 **2.2. Treatment of Hyperthyroidism and Other Benign Thyroid Conditions**

284 (15) ^{131}I -iodide, first used in the 1940s (Seidlin et al., 1946), is a routine treatment for
285 diffuse or nodular toxic goitre, hyperthyroidism, or large non-toxic goitre (Leiter et al., 1946).
286 The treatment is usually performed by oral administration of a capsule containing ^{131}I -iodide,
287 but ^{131}I solution is also used for individualized administration of the activity. Radioactive
288 iodine accumulates in the thyroid gland, and beta particles emitted by ^{131}I destroy the cells of
289 the thyroid gland. Although this is firmly established as a first line treatment, there is little
290 consensus concerning treatment regimens, and ongoing controversy over the aims of
291 treatment.

292 **2.2.1. Aim of treatment**

293 (16) The aim of treatment is to destroy the thyroid glands' cells and suppress the
294 hyperactive thyroid function to render the patient euthyroid or in a hypothyroid state.

295 **2.2.2. Treatment protocols**

296 (17) Treatment protocols fall into 3 categories according to the purpose of treatment:
297 - An administration of a fixed activity with an aim to render patients hypothyroid within a
298 short period of time, whereupon patients continue on life-long thyroid replacement
299 hormones (Royal College of Physicians, 2007).

- 300 - A personalised approach to inducing hypothyroidism, in order to achieve a swift
301 response albeit with the minimal administered activity necessary (Kobe et al., 2008;
302 Stokkel et al., 2010; Schiavo et al., 2011).
- 303 - A personalised approach to treatment with the aim of rendering patients euthyroid where
304 possible, to delay the need for supplementary medication (Flower et al., 1994; Howarth
305 et al., 2001).

306 **2.2.3. Radiation dose to friends and family**

307 (18) Radioiodine is primarily excreted via urine, but also through faeces and perspiration
308 (Hänscheid et al., 2013; ICRP, 2015a, 2015b). The mean effective half-life for excretion of
309 ^{131}I from the thyroid is about 5 days, although this has been shown to vary widely.
310 Assessments should be performed for individual treatments, taking into account patient-
311 specific circumstances and detailed written instructions, and written guidance should be
312 provided to the patient and their family.

313 **2.2.4. Radiation dose to staff and carers**

314 (19) The levels of activity administered for treatments of benign thyroid conditions are
315 often substantially less than those administered for ablation or therapy procedures, although
316 they are commonly greater than those administered for diagnostic studies. Effective dose
317 estimates for staff members are therefore necessary for administration procedures, and there
318 may also be a need to follow the thyroid doses for those working with ^{131}I . Precautions must
319 be taken considering time, distance and shielding, and to avoid contamination. Comforter and
320 carer consent is required if in close contact with the patient.

321 **2.2.5. Patient organ dosimetry**

322 (20) The role of internal dosimetry in the management of benign thyroid disease with
323 radioiodine remains a matter of debate. In some cases fixed activities are administered while
324 in others, dosimetry is routinely performed and may be used to guide treatment. A number of
325 methods have been employed (Stokkel et al., 2010). Advances in quantitative imaging and
326 dosimetry enable more precise dosimetry calculations that may take into account volume and
327 sequential retention measurements acquired from ^{131}I or ^{123}I SPECT, MRI, and ^{124}I PET. The
328 accuracy and reproducibility of internal dosimetry have been subject to increased
329 investigation and should be further developed (Metso et al., 2007; Merrill et al., 2011) when
330 reporting absorbed doses. Dosimetry guidelines have been published by the European
331 Association of Nuclear Medicine (EANM) (Hänscheid et al., 2013).

332 **2.2.6. Risks to patients**

333 (21) As with all therapeutic procedures, pregnancy and breastfeeding are a
334 contraindication to treatment and patients should avoid conception for 4-6 months, dependent
335 on national guidelines. Patients to be treated with radioactive iodine should not undergo tests
336 with iodinated contrast media within two months prior to the therapy due to the risk of iodine
337 blockage with low uptake of radioactive iodine (Luster et al., 2008).

338 **2.2.7. Recommendations**

339 (22) At present there are no standardised protocols for treatment, which reflects the lack
340 of evidence base for best practice. There is evidence that a fixed activity administration,
341 without dosimetry calculations, while convenient for many centres, results in the
342 administration of higher activities than is necessary, in contravention of the ALARA
343 principle (Jönsson and Mattsson, 2004; Sisson et al., 2007).

344 (23) In principle, a fully personalised approach, based on patient-specific measurements
345 can ensure the administration of a minimal effective activity, thereby minimising the
346 potential for long term risks and the radiation doses delivered to staff, family and comforters
347 and carers. Of particular importance to this treatment, a personalised approach also offers the
348 potential to render patients euthyroid where this may be desired and reports have indicated
349 that such an approach is possible, at least in a subset of patients. There have been a limited
350 number of trials to date to investigate the potential of a personalised approach to treatment
351 (Leslie et al., 2003) and further trials are needed to determine the relationship between the
352 absorbed doses delivered to the thyroid and to normal organs and outcome. Such trials should
353 be stratified according to the volume of the thyroid, initial uptake and retention as there is
354 some evidence that these may be confounding factors (Howarth et al., 2001; Reinhardt et al.,
355 2002).

356 **2.3. Treatment of Differentiated Thyroid Cancer**

357 (24) ¹³¹I-iodide, first used in the 1940s (Seidlin et al., 1946), has become a treatment of
358 choice for the ablation and therapy of papillary and follicular thyroid cancer. Patients are
359 typically given a low iodine diet prior to administration (Haugen et al., 2016). Some
360 guidelines now also indicate the use of recombinant human thyroid stimulating hormone
361 (rhTSH; Thyrogen, Genzyme Corp.) as an adjunctive treatment to stimulate uptake for
362 radioiodine ablation of thyroid tissue remnants in patients who have undergone near-total or
363 total thyroidectomy for well-differentiated thyroid cancer and who have evidence of distant
364 metastatic thyroid cancer. Subsequent administrations are given for further therapy of
365 recurrent or persistent disease, particularly in the case of metastatic spread. Administrations
366 are continued, typically at 6-8 month intervals, until patients become iodine negative or fail to
367 show response.

368 (25) Management guidelines have been published for adult patients with thyroid nodules
369 and differentiated thyroid cancer (Haugen et al., 2016) and for the diagnosis and management
370 of thyroid disease during pregnancy and the postpartum period (Alexander et al., 2017).
371 However, practical guidelines for therapy of thyroid disease with ¹³¹I still vary and are
372 increasingly based on patient preferences (Silberstein et al., 2012).

373 **2.3.1. Aim of treatment**

374 (26) For ablation, the aim of treatment is to eradicate residual thyroid tissue. With further
375 therapy, the aim of treatment is to eradicate malignant tissues. Several professional medical
376 societies have provided management guidelines for patients with thyroid nodules and
377 differentiated thyroid cancer (Luster et al., 2008; Haugen et al., 2016). For some staging
378 criteria, there are uncertainties over the potential usefulness of radioiodine (Perros et al.,
379 2014). In some cases, for those concerning children and young people, persistent yet stable
380 disease is expected.

381 2.3.2. Treatment protocols

382 (27) In spite of the widespread use of this treatment over many decades, the level of
383 evidence for optimal radioiodine treatments is extremely low (Luster et al., 2008). No
384 multicentre trials have yet been conducted to establish the optimal activity to administer for
385 either ablation or for subsequent therapeutic procedures. Consequently guidelines do not give
386 recommendations regarding levels of administration, and such recommendations that are
387 provided are necessarily based on expert advice.

388 (28) In recent years the UK HiLo trial and the French ESTIMABL trial demonstrated that
389 1.1 GBq is as effective as 3.7 GBq for ablation in low or intermediate risk patients, although
390 the interpretation of these results is debated. There is ongoing discussion as to whether
391 radioiodine should be administered at all in low risk patients (Mallick et al., 20012b;
392 Schlumberger et al., 2012; Haugen et al., 2016).

393 (29) In the absence of trial-based evidence, activity schedules have been proposed to
394 minimise the likelihood of secondary malignancies although a 'safe' level of activity is yet to
395 be determined. There is no evidence to suggest whether interval or high single treatments are
396 optimal for response and toxicity measurements.

397 (30) To date there have been no randomised controlled clinical trials for the treatment of
398 children with differentiated thyroid cancer, and only one set of guidelines have been
399 produced (Francis et al., 2015). Administrations for radioiodine ablation in children vary
400 widely. Activity may be adjusted by body weight (usually 1.85–7.4 MBq kg⁻¹), by body
401 surface area or by age (Jarzab et al., 2005; Luster et al., 2008). A hybrid approach of
402 combining 24-hour-uptake measurements with body weight is favoured by the German
403 procedure guidelines (Franzius et al., 2007).

404 (31) Treatment protocols for therapy administrations also vary. Fixed activities of 1.1
405 GBq – 11.0 GBq have been administered to children, as well as a range of activities based on
406 body weight (Jarzab et al., 2005; Franzius et al., 2007; Luster et al., 2008; Verburg et al.,
407 2011).

408 2.3.3. Radiation dose to friends and family

409 (32) Radioiodine is primarily excreted via the urine, but also through faeces and
410 perspiration (Hänscheid et al., 2013; ICRP, 2015a, 2015b). The mean effective half-life for
411 excretion after total thyroidectomy is much less than that with hyperthyroidism (Hänscheid et
412 al., 2006; Remy et al., 2008). Assessments should be performed for individual treatments,
413 taking into account patient-specific circumstances and detailed written instructions, and
414 written guidance should be provided to the patient and their family. Comforter and carer
415 consent is required if in close contact with the patient.

416 (33) Patients undergoing treatment may require hospitalisation for a number of days
417 following administration, according to national regulations. The decision to hospitalise or to
418 release a patient should be determined on an individual basis, and the time of release judged
419 by monitoring dose rate from the patient to assess the residual activity in the patient.

420 2.3.4. Radiation dose to staff and carers

421 (34) As with all procedures involving radiotherapeutics, standard precautions should be
422 taken with the principle of ALARA. As patients are hospitalised, there are risks to different

423 groups of staff, including nurses, technologists, physicists and physicians, and staff doses
424 should be monitored.

425 **2.3.5. Patient organ dosimetry**

426 (35) Fixed administration protocols result in the delivery of a very wide range of
427 absorbed doses (Flux et al., 2010). The use of dosimetry in radioiodine treatment of thyroid
428 cancer to develop personalised treatment planning is increasing.

429 (36) Of note, dosimetry was performed at the outset by Seidlin et al. (1946) to calculate
430 the cumulative absorbed doses delivered to metastases. Further influential studies have
431 included the establishment in 1962 of a blood absorbed dose of 2 Gy as a surrogate biomarker
432 for marrow toxicity (Benua et al., 1962) and a figure of 300 Gy to ablate thyroid remnant
433 tissue and 80 Gy to eradicate lymph node metastases (Maxon et al., 1992).

434 (37) Since that time, a number of dosimetry studies have been performed that, although
435 giving some variation in absolute values, have nevertheless shown significant correlations
436 between the absorbed doses delivered and response (Strigari et al., 2014) and dosimetry
437 guidelines have been published by the EANM (Lassmann et al., 2008).

438 **2.3.6. Risks to patients**

439 (38) As with all therapy procedures, pregnancy/breastfeeding is a contraindication, and
440 patients should avoid conception. A range of side effects can arise from administration of
441 radioiodine, the most common being sialadenitis and gastritis (Luster et al., 2008). A single
442 administration of radioiodine can induce permanent xerostomia and can increase the risk of
443 salivary malignancies (Klubo-Gwiezdzinska et al., 2010; Lee, 2010). A decline in leucocytes
444 and platelets may also be seen and there are risks of pulmonary fibrosis in patients with lung
445 metastases (Haugen et al., 2016). Patients to be treated with radioactive iodine should not
446 undergo tests with iodinated contrast media within two months prior to the therapy due to the
447 risk of iodine blockage with low uptake of radioactive iodine (Luster et al., 2008).

448 (39) Children and young people treated with radioiodine for differentiated thyroid cancer
449 are likely to have a significantly longer survival than is the case for adults, although 2% have
450 long term cause-specific mortality. Many children with pulmonary metastases develop stable
451 disease following administration of radioiodine (Vassilopoulou-Sellin et al., 1993; Pawelczak
452 et al., 2010). Long term follow up of children treated with radioiodine for differentiated
453 thyroid cancer has shown an increase in secondary malignancies (Rubino et al., 2003; Brown
454 et al., 2008; Hay et al., 2010; Francis et al., 2015). The risk of leukaemia increases with
455 increasing cumulative activity and patients are more likely to develop secondary
456 malignancies in the bladder, colorectal system, breast and salivary glands. Decreased
457 spermatogenesis can also result from increasing cumulative activities of radioiodine which
458 can have long term consequences for survivors.

459 **2.3.7. Recommendations**

460 (40) The overall cause-specific survival for differentiated thyroid cancer is approximately
461 85% (Luster et al., 2008). However, it is possible that this figure is heavily influenced by the
462 number of low risk patients (where risk may be generally defined according to a number of
463 factors including age, volume of disease and metastatic spread) that may not in fact require an
464 administration of radioiodine (Mallick et al., 2012a). Survival for high risk patients including

465 those with metastases is only 25 – 40 %, indicating the need for stratification in treatment
466 planning. Also of note is that the recurrence rate can be as high as 10 – 30 %. Insufficient
467 treatment will entail further therapy at the risk of continuing progression and the development
468 of iodine negativity. Excess radiation delivered to normal tissues is associated with potential
469 side effects and some risk of secondary malignancies.

470 (41) While it may be argued that the low mortality precludes the necessity to optimise
471 treatments further, the obvious benefit of living without disease, and the need to minimise the
472 potential for secondary malignancies are strong arguments for the consideration of dosimetry
473 for each treatment following therapy. This is particularly relevant for children and young
474 people, and for high risk patients. Further studies are needed to investigate the role of pre-
475 therapy dosimetry planning, taking into account the possibility of stunning, whereby uptake
476 of activity for therapy is reduced. Thyroid stunning is a clinical problem in which exposure of
477 a patient to diagnostic amounts of ^{131}I has been described to alter the ability of differentiated
478 thyroid carcinoma, or remnants of thyroid tissue after thyroidectomy, to take up therapeutic
479 amounts of ^{131}I .

480 **2.4. Treatment of Polycythaemia Vera and Essential Thrombocythaemia**

481 (42) ^{32}P phosphate was first used to treat polycythaemia vera (PV) and essential
482 thrombocythaemia (ET) about 70 years ago. PV and ET are chronic progressive
483 myeloproliferative disorders characterised by an over-production of erythrocytes and
484 thrombocytes, respectively. Other disease features include leucocytosis, splenomegaly,
485 thrombohaemorrhagic complications, vasomotor disturbances, pruritus, and a risk of disease
486 progression into acute myeloid leukaemia or myelofibrosis. With the introduction of agents
487 such as hydroxycarbamide, interferon and anagrelide, the role of ^{32}P has diminished. Today,
488 PV and ET remain the only myeloproliferative conditions in which ^{32}P is indicated.

489 **2.4.1. Aim of treatment**

490 (43) ^{32}P is actively incorporated into DNA of rapidly proliferating cells and the treatment
491 suppresses the blood cell production by irradiation of the bone marrow. The
492 radiopharmaceutical is used to suppress hyper-proliferative cell lines rather than to eradicate
493 them. In spite of there being a number of alternative treatments, there remains a subgroup of
494 elderly patients with PV and ET for whom ^{32}P as orthophosphate is used orally or by
495 intravenous injection (Tennvall and Brans, 2007).

496 **2.4.2. Treatment protocols**

497 (44) The radiopharmaceutical is administered intravenously or orally. The activity
498 generally used is either 74–111 MBq m^{-2} body surface with a maximum upper activity limit
499 of 185 MBq, or a slightly higher activity of 3.7 MBq kg^{-1} body weight with a maximum
500 upper activity limit of 260 MBq. A decrease in activity of 25% in patients >80 years of age is
501 recommended by some investigators. An alternative, dose-escalating approach is to
502 administer a fixed lower activity of 111 MBq. In the absence of an “adequate response”, a
503 second treatment is to be given after 3 months, this time with a 25% increase in activity. This
504 procedure of increased activity may be repeated every 3 months until an adequate response is

505 obtained. The upper activity limit for a single administration is 260 MBq (Tennvall and Brans,
506 2007).

507 **2.4.3. Radiation dose to friends and family**

508 (45) For outpatient therapy, there is a need for instructions to patient and family
509 indicating 1) the need to avoid prolonged, close contact with young children and pregnant
510 women, 2) a recommendation to sleep in a separate bed from partner or children for a few
511 days after return home, 3) the need for personal hygiene to avoid any external contamination
512 (^{32}P is excreted in urine for a period of two to three weeks).

513 **2.4.4. Radiation dose to staff and carers**

514 (46) As ^{32}P is a high energy beta emitter, it is essential to shield with PMMA during
515 dispensing and injection.

516 **2.4.5. Patient organ dosimetry**

517 (47) Organs with the highest radiation absorbed dose are bone endosteum and
518 haematopoietically active bone marrow, receiving around 11 mGy per MBq administered. A
519 typical administration of 100 MBq thus gives over 1 Gy to bone endosteum and active bone
520 marrow.

521 **2.4.6. Risks to patients**

522 (48) Contraindications are pregnancy and breastfeeding, and patients should avoid
523 conception. The radiopharmaceutical is not recommended for women of childbearing age.
524 The incidence of acute myeloid leukaemia (AML) 10 years after ^{32}P treatment was
525 approximately 10% (Brandt and Anderson, 1995). Treatment using ^{32}P is therefore usually
526 reserved for patients over the age of 65 – 70 years.

527 **2.4.7. Recommendations**

528 (49) ^{32}P can be used in elderly patients and those for whom alternative treatments using
529 e.g. hydroxyurea, busulphan, interferon-alpha or anagrelide are not suitable.

530 **2.5. Treatment of Skeletal Metastases**

531 (50) Treatment of pain that is derived from skeletal metastases is one of the important
532 issues in the management of cancer patients who are in advanced stages and need palliative
533 care. Painful bone metastases may impair quality of life through limitation of daily activity,
534 restricted mobility, insomnia, and anxiety. Management of bone pain should be
535 multidisciplinary, involving analgesia, radiation, hormones, chemotherapy, bisphosphonates,
536 and surgery. Localised metastases can be treated with external beam radiation, or surgery,
537 whereas more diffuse bone metastases are usually treated by radiopharmaceuticals, hormones,
538 chemotherapy, and bisphosphonates (Pandit-Taskar et al., 2004).

539 (51) Radiopharmaceuticals that emit beta particles such as ^{89}Sr chloride and ^{153}Sm -
540 EDTMP (ethylenediamine tetramethylene phosphonate) have been administered for pain

541 relief in patients with painful skeletal metastases as palliative therapy. ^{223}Ra -dichloride, an
542 alpha-emitting bone-seeking radiopharmaceutical, has appeared as a curative
543 radiopharmaceutical therapy agent for castration-resistant prostate cancer with symptomatic
544 bone metastases and has been shown to prolong overall survival (by approximately 3 months)
545 in comparison with a placebo (Parker et al., 2013; Pandit-Takar et al., 2014).

546 **2.5.1. Aim of treatment**

547 (52) The aim of treatment with beta emitting radiopharmaceuticals is to control bone pain
548 due to metastases and to improve quality of life in patients suffering from malignancies. The
549 aim is principally palliation and anticancer effects that produce survival benefits are usually
550 not evident. ^{89}Sr -chloride and ^{153}Sm -EDTMP are approved in a number of nations for the
551 palliation of pain due to skeletal metastases from solid cancers, while ^{186}Re -HEDP
552 (hydroxyethylenediphosphonate), ^{188}Re -HEDP, $^{117\text{m}}\text{Sn}$ -DTPA
553 (diethylenetriaminepentaacetic acid), and ^{177}Lu -EDTMP are under investigation (Finlay et al.,
554 2005; Liepe et al., 2005b, 2007; Shinto et al., 2014; Yousefnia et al., 2015). The mechanism
555 of pain relief by these radiopharmaceuticals is not fully understood. The aim of treatment of
556 ^{223}Ra -dichloride therapy is to provide prolonged overall survival in castration-resistant
557 prostate cancer patients with bone metastases.

558 **2.5.2. Treatment protocols**

559 (53) ^{89}Sr -chloride and ^{153}Sm -EDTMP have approval in a number of nations and thus have
560 well-established treatment protocols. ^{89}Sr -chloride at an activity per body weight of 1.5 - 2.2
561 MBq kg⁻¹ body weight is administered at a single intravenous injection as compared to ^{153}Sm -
562 EDTMP at an activity per body weight of 37 MBq kg⁻¹. For both radiopharmaceuticals,
563 patients have to visit their doctors regularly to ensure that the treatments are working and to
564 check for unwanted effects including leukocytopenia and thrombocytopenia. Treatment
565 protocols are under study for ^{186}Re -HEDP, ^{188}Re -HEDP, $^{117\text{m}}\text{Sn}$ -DTPA, and ^{177}Lu -EDTMP
566 (Pandit-Taskar et al., 2004; Liepe and Kotzerke, 2007; Bodei et al., 2008; D'Angelo et al,
567 2012; Jie et al., 2013; Thapa et al., 2015).

568 (54) The approved administered activity per body weight for ^{223}Ra -dichloride is 55 kBq
569 kg⁻¹ given intravenously as 6 administrations every 4 weeks.

570 **2.5.3. Radiation dose to friends and family**

571 (55) As activity is excreted mainly through urine for ^{89}Sr -chloride and ^{153}Sm -EDTMP
572 and through faeces for ^{223}Ra -dichloride, care must be taken to ensure that all excreta are
573 disposed of in the sanitary sewer system when a patient is at home. Patients may be
574 hospitalised for longer time if mentally incompetent and/or incontinent and therefore
575 incapable of following radiation safety instructions and precautions (ICRP, 2004).

576 **2.5.4. Radiation dose to staff and carers**

577 (56) For ^{89}Sr , ^{153}Sm -EDTMP and ^{223}Ra patients can receive treatment on an outpatient
578 basis, which is advantageous for ensuring that exposures of staff remain within acceptable
579 limits. Higher irradiation of ^{186}Re -HEDP and ^{188}Re -HEDP results from the gamma emissions.
580 Staff doses should be carefully monitored in all cases. ^{223}Ra -dichloride has been evaluated as

581 safe and straightforward to administer using conventional nuclear medicine equipment
582 (Dauer et al. 2014).

583 **2.5.5. Patient organ dosimetry**

584 (57) ^{89}Sr gives absorbed doses of 0.2 - 2 and 0.05 – 0.3 Gy MBq⁻¹ to the metastatic sites
585 and red marrow, respectively (Breen et al., 1992), while $^{153}\text{Sm-EDTMP}$ gives absorbed doses
586 of 5.3-8.8 and 1.2-2.0 mGy MBq⁻¹ to the bone surfaces and red marrow, respectively (Eary et
587 al., 1993). Absorbed dose values may vary depending on dosimetric models and
588 biodistribution data. Of particular interest is the uncertainty over radiation weighting values
589 for alpha dosimetry that can vary from 3-5 for deterministic effects (as is the case for
590 radiotherapy) and for stochastic effects is recommended as 20 by the ICRP (Sgouros et al.,
591 2010; Lassmann and Nosske, 2013).

592 **2.5.6. Risk to patients**

593 (58) Radiopharmaceuticals used for therapy of bone metastases must be used carefully as
594 they may cause bone marrow suppression, especially in patients with reduced bone marrow
595 reserve who have previously been treated with repeated chemotherapy. A transient rise in
596 bone pain (flare) a few days after administration is recorded in some patients but usually not
597 severe. Patients with renal dysfunction must undergo a careful evaluation prior to treatment
598 because adverse effects including bone marrow suppression may be more serious.
599 Contraindications are pregnancy and breastfeeding, and patients should avoid conception.

600 (59) ^{223}Ra has the advantage of sparing much of the marrow from irradiation, given the
601 short-range alpha emissions. Non-haematological toxicities are generally more common than
602 haematologic toxicity and are mild to moderate in intensity. The most common side effects
603 are diarrhoea, fatigue, nausea, vomiting, and bone pain, some of which are dose-related.
604 These side effects are easy to manage, and treatment is symptomatic and supportive (Pandit-
605 Taskar et al., 2014). The long-term effects of ^{223}Ra in patients with extended survival are not
606 yet known.

607 **2.5.7. Recommendations**

608 (60) Bone seeking radiopharmaceuticals have important roles in the management of
609 painful bone metastases by alleviating pain and improving quality of life. Pain relief may last
610 several months after a single injection of radiopharmaceuticals. The widely different
611 administration protocols for each agent, that may be fixed or weight-based and may be
612 administered once or multiple times, indicates that optimal treatment protocols are not yet
613 established and further studies are necessary to this end. In terms of adverse effects,
614 haematological toxicity due to marrow exposure should be taken into account. An
615 investigation of the optimal absorbed dose to deliver for ^{223}Ra would help to determine
616 optimal treatment regimens and to identify patients in whom treatment is likely to have little
617 or no benefit. The radiopharmaceuticals are administered usually on an outpatient basis and
618 standard radiation protection precautions are required.

619

620 **2.6. Treatment of Neuroblastoma in Children and Young Adults**

621 (61) Metaiodobenzylguanidine (mIBG), introduced in the 1980s, is a guanethidine and
622 noradrenaline analogue taken up by cells of the sympathetic nervous system by an active
623 transport process involving the noradrenaline transporter molecule.

624 (62) Neuroblastoma arises from the neural crest cells involved in the development of the
625 nervous system and other tissues. It commonly occurs in the adrenal glands or in the nerve
626 tissue and can spread to bones and liver. It accounts for around 6% of childhood cancers with
627 only 67% surviving 5 years. ¹³¹I-mIBG is most commonly administered in chemo-refractory
628 or relapsed patients. Outcome is variable with response varying from 30% - 58% (Hoefnagel
629 et al., 1991; Garaventa et al., 1999; Matthay et al., 2007).

630 **2.6.1. Aim of treatment**

631 (63) The aim of treatment is predominantly palliative. A range of responses are seen,
632 including complete responses and downstaging, which may permit further surgery or external
633 beam radiotherapy (George et al., 2016).

634 **2.6.2. Treatment protocols**

635 (64) Treatment regimens for ¹³¹I-mIBG therapy for neuroblastoma vary widely. There are
636 currently no established guidelines to govern the levels of activity administered. Typically,
637 empirical fixed activities have been administered, comprising multiples of 3.7 GBq
638 (Hoefnagel et al., 1991; Tristram et al., 1996), although weight-based activities have also
639 frequently been administered. There is evidence that short term toxicity is significantly
640 correlated with the whole-body absorbed dose, which can therefore act as a surrogate for the
641 absorbed dose delivered to the red marrow. This has led to an alternative approach to fixed
642 activity administrations, whereby the activities are tailored to deliver a prescribed whole-
643 body absorbed dose (Gaze et al., 2005; Buckley et al., 2009). This can entail two
644 administrations of 555 – 666MBq kg⁻¹ to deliver a total whole body absorbed dose of 4 Gy,
645 with peripheral blood stem cell support (Giammarile et al., 2008). There is, similarly, no
646 protocol to govern the number of treatments delivered, and although single treatments have
647 been administered, these are typically repeated once or twice. However, as many as five
648 administrations have been reported (George et al., 2016).

649 **2.6.3. Radiation dose to friends and family**

650 (65) Individual risk estimates must be performed for each patient, taking home
651 circumstances into account. This is particularly relevant for children and young people that
652 may have siblings at home. Excretion is predominantly via the urine, and care must be taken
653 to ensure that all excreta are disposed of in the sanitary sewer system. Written instructions
654 must be provided to patients and to their families/carers on discharge.

655 **2.6.4. Radiation dose to staff and carers**

656 (66) Careful protection procedures are required to minimise radiation from the source
657 and the administered patient. Shielded syringes should be utilised during the intravenous
658 administration to ensure that extremity doses are maintained below occupational dose
659 constraints. The use of automatic injection system will significantly reduce the effective dose

660 to the staff members (Rushforth et al., 2017). Administration protocols must be carefully
661 considered. Personalised protocols (Gaze et al., 2005; Buckley et al., 2009) can entail
662 extremely high levels of radiation in comparison with other treatments and as patients may be
663 very young, a high level of care is required. Nursing staff in particular require specific
664 training. Valuable advice related to administration of high-dose ¹³¹I-MIBG therapy to
665 children is given by Chu et al. (2016).

666 **2.6.5. Patient organ dosimetry**

667 (67) In contrast to many therapy procedures with radiopharmaceuticals, a large number of
668 dosimetry studies have been performed relative to the number of centres that offer this
669 treatment (Tristram et al., 1996; Matthay et al., 2001; Sudbrock et al., 2010; Flux et al., 2011).
670 The absorbed doses delivered to whole-body, critical organs and tumours have been reported
671 to vary by an order of magnitude (Matthay et al., 2001; Flux et al., 2011).

672 **2.6.6. Risks to patients**

673 (68) Acute toxicity is primarily haematological, causing neutropenia, thrombocytopenia
674 and leukocytopenia (Buckley et al., 2009). Thyroid blockade is essential, but hypothyroidism
675 can nevertheless result in over 10% of cases and hepatic toxicity has been reported in 75% of
676 patients (Quach et al., 2011). Secondary malignancies have been reported in up to 5% of
677 cases (Weiss et al., 2003).

678 **2.6.7. Recommendations**

679 (69) Although patients frequently present with advanced disease, long-term survival is
680 not uncommon. The probability of inducing acute myelotoxicity, the potential for longer-term
681 secondary neoplasms and the need to justify administrations of high activity to children and
682 young people emphasise the need for personalised dosimetry planning and verification for all.

683 **2.7. Treatment with Radiolabelled Peptide Receptor**

684 (70) Neuroendocrine tumours express somatostatin receptors (SSR). Radiolabelled
685 analogues of somatostatin have been developed for therapeutic purposes including ⁹⁰Y-
686 DOTATOC ([⁹⁰Y-DOTA⁰,Tyr³]-octreotide) and ¹⁷⁷Lu-DOTATATE ([¹⁷⁷Lu-
687 DOTA⁰,Tyr³,Thr⁸]-octreotide or [¹⁷⁷Lu-DOTA⁰,Tyr³]-octreotate) that target the somatostatin
688 receptor subtype 2. To date, a lack of randomised clinical trials has precluded evidence-based
689 guidelines, although limited guidelines have been produced (Ramage et al., 2012) and a
690 guidance document has been published jointly by the IAEA, EANM and SNMMI based
691 predominantly on expert opinion (Bodei et al., 2013).

692 (71) The ideal radionuclide has not been established and there are arguments to support
693 both ⁹⁰Y and ¹⁷⁷Lu. ⁹⁰Y, with a substantially longer range of beta-particles, is more able to
694 deposit a uniform distribution of energy at a multicellular scale in the event of heterogeneous
695 uptake, whereas it has been argued that this can produce greater kidney toxicity due to
696 irradiation of the cortex (Bodei et al., 2008). ¹⁷⁷Lu also has the advantage of quantitative
697 imaging for dosimetry, whereas a ⁹⁰Y administration must be ‘spiked’ with a tracer level of
698 ¹¹¹In. The physical half-lives of both radionuclides (64 hours and 6.7 days for ⁹⁰Y and ¹⁷⁷Lu
699 respectively) are compatible with the biological retention following uptake and do not cause

700 unnecessary hospitalisation. Both ^{177}Lu DOTATATE and ^{90}Y DOTATATE are radiolabelled
701 in house, necessitating the usual precautions for such procedures.

702 **2.7.1. Aim of treatment**

703 (72) Response is variable and the aims of treatment are predominantly palliative. Partial
704 or complete objective responses have been reported in up to 30 % of patients; in particular
705 complete responses have been reported in 2-6% of patients with gastroenteropancreatic
706 tumours (Bodei et al., 2013). Treatments are administered to adults, although one clinical trial
707 has investigated the potential of ^{177}Lu -DOTATATE treatment of children and young people
708 with neuroblastoma (Gains et al., 2011).

709 **2.7.2. Treatment protocols**

710 (73) Treatment protocols have become to a limited extent standardised based on
711 established practice. There are nevertheless variations. ^{90}Y -DOTATATE or ^{90}Y -DOTATOC
712 is frequently administered as 3.7 GBq m^{-2} body surface for 2 cycles or with a fixed activity of
713 2.78 – 4.44 GBq for 2-4 cycles. ^{177}Lu -DOTATATE is commonly administered as a fixed
714 activity of 5.55 - 7.4 GBq over 3-5 cycles. The interval between administrations varies from 6
715 – 12 weeks (Bodei et al., 2013). Patients with compromised renal function are recommended
716 to be administered lower activities. Patients with compromised marrow reserves may require
717 a stem cell harvest for subsequent reinfusion although haematological toxicity is generally
718 very low. Combination therapies of ^{90}Y - and ^{177}Lu -DOTATATE administered alternately are
719 currently under investigation (Kunikowska et al., 2011; Savolainen et al., 2012; Seregni et al.,
720 2014). There have been no activity or absorbed dose escalation trials to establish optimal
721 administration protocols, either at a population level or for individual patients.

722 (74) There are high levels of somatostatin receptors in children and young people with
723 neuroendocrine tumours, although with few exceptions clinical trials exclude this patient
724 population due to unknown safety profile (Menda et al., 2010; Schmidt et al., 2010; Gains et
725 al., 2011).

726 **2.7.3. Radiation dose to friends and family**

727 (75) Activity is excreted through body fluids, primarily urine and perspiration. Care must
728 therefore be taken when a patient is discharged, and home circumstances should be taken into
729 account.

730 **2.7.4. Radiation dose to staff and carers**

731 (76) Patients are typically hospitalised for one or two nights only which entails risks of
732 exposures of different groups of staff, including nurses, technologists, physicists and
733 physicians. For the treatment of beta particle emitting radionuclides, including ^{90}Y and ^{177}Lu ,
734 particular attention should be taken for the staff working on preparation and handling of
735 radiopharmaceuticals given to the patient. Shielded syringes should be utilised during the
736 intravenous administration of radiopharmaceuticals as necessary to ensure that extremity
737 doses are maintained below occupational dose constraints. Doses to the finger tips from
738 preparation and administration are typically in the range 5-10 mSv from single
739 administrations when protection is optimised, but can be over 100 mSv if precautions are
740 inadequate. Monitoring the dose to the finger tips using finger stall dosimeters for the main

741 fingers carrying out manipulations is strongly recommended for radiological protection in
742 order to give a realistic picture of staff dose levels (Cremonesi et al., 2006b; ICRP, 2008;
743 Grassi et al, 2009; Vanhavere et al., 2012).

744 **2.7.5. Patient organ dosimetry**

745 (77) Internal dosimetry is employed routinely in only a minority of centres and may be
746 applied to tumours and to organs-at-risk including kidney and liver. Active marrow absorbed
747 doses per administered activity from ^{90}Y -DOTATATE have been reported ranging from 0.03
748 – 0.17 Gy GBq⁻¹, kidney absorbed doses from 1.71 – 2.73 Gy GBq⁻¹ and liver absorbed doses
749 from 0.27 – 0.92 Gy GBq⁻¹ (Cremonesi et al., 2006a, 2010; Bodei et al., 2008). Absorbed
750 doses per administered activity from ^{177}Lu -DOTATATE to active marrow, kidneys and liver
751 have been reported as ranging from 0.02 – 0.07 Gy GBq⁻¹, 0.32 – 1.67 Gy GBq⁻¹ and 0.05 –
752 0.21 Gy GBq⁻¹ respectively. Although correlations between absorbed dose and effect have
753 not been an endpoint of any clinical trial to date, there is increasing evidence of such
754 correlations covering both response (Pauwels et al., 2005; Ilan et al., 2015) and toxicity
755 (Barone et al., 2005; Walrand et al., 2011; Strigari et al., 2014). There is evidence that the
756 intra-patient variation in absorbed doses is small, whereas the inter-patient variation is
757 significant (Hindorf et al., 2007; Sundlöv et al., 2017).

758 **2.7.6. Risks to patients**

759 (78) As with all therapy procedures, pregnancy/breastfeeding is a contraindication, and
760 patients should avoid conception. Excretion is predominantly urinary and, hence, amino acids
761 are routinely co-administered to protect kidneys. Kidney toxicity is nevertheless seen (Barone,
762 2005; Imhof, 2011) and a biologically effective dose (BED) of < 28 Gy (see section 4.7) has
763 been recommended for patients with higher risk factors treated with ^{90}Y -DOTATATE (Bodei
764 et al., 2008). A corresponding value for patients treated with ^{177}Lu -DOTATATE has yet to be
765 determined. Grade 3-4 myelotoxicity is observed in up to 10-13% of patients and cases of
766 myelodysplastic syndrome or overt acute myelogenous leukaemia have been reported
767 (Valkema et al., 2002; Barone et al., 2005; Kwekkeboom et al., 2005; Bushnell et al., 2010;
768 Strosberg et al., 2017).

769 **2.7.7. Recommendations**

770 (79) The treatment of adult and paediatric neuroendocrine cancers with radiolabelled
771 peptides continues to develop and expand. As yet, there are few data to inform long term risk
772 estimates although there is abundant evidence for acute toxicity primarily to kidneys and to
773 bone marrow. The inter-patient variation in absorbed doses delivered to tumours and the
774 potential for acute radiation induced nephrotoxicity and myelosuppression mean that
775 prospective patient-specific organ and tissue dosimetry should ideally be performed for all
776 patients. This may not always be feasible in which case, as treatment is almost invariably
777 administered in multiple cycles, an initial administration according to a fixed activity or body
778 surface area can safely establish the biokinetics of the individual patient. Retrospective
779 dosimetry should be performed before and following subsequent administrations which may
780 then be modified according to the cumulative absorbed doses delivered to tumours and
781 organs-at-risk. The prospect of personalised treatments based on carefully designed
782 dosimetry protocols is quite feasible. There is some evidence that biological parameters such

783 as BED can be of benefit to calculate risks of toxicity to organs at risk (OARs) and these
784 should be further investigated and considered (Barone et al., 2005; Wessels et al., 2008).

785 **2.8. Radioimmunotherapy**

786 (80) Radioimmunotherapy involves radiolabelled antibodies that recognise tumour-
787 specific antigens and deliver therapeutic radiation to neoplasms (Barbet et al., 2012).
788 Antibodies may be mouse monoclonal antibodies, or in many cases human/mouse chimeric
789 or humanised antibodies that are obtained by genetic engineering technologies in order to
790 reduce immunogenicity in humans. Mostly radionuclides are beta emitters such as ^{131}I , ^{90}Y ,
791 ^{186}Re , and ^{153}Sm , and lately alpha emitters such as ^{225}Ac and ^{213}Bi are also recognised as
792 potentially useful, and have been used in humans in some preliminary clinical studies
793 (Sgouros et al., 2010; Larson et al., 2015).

794 (81) Substantial efforts have focused on research for development of
795 radioimmunotherapy although to date only two agents have been approved by health
796 authorities as commercially available radioimmunotherapy agents; ^{131}I -tositumomab and ^{90}Y -
797 ibritumomab tiuxetan (Goldsmith, 2010). Both are directed to CD-20 positive, relapsed or
798 refractory, low-grade or follicular B-cell non-Hodgkin's lymphoma and provide high
799 response rate, although sufficient long-term survival data have not yet been accumulated.
800 ^{90}Y -ibritumomab tiuxetan is effectively applied also to consolidation therapy, that is, therapy
801 for patients with previously untreated lymphoma who achieve a partial or complete response
802 to first-line chemotherapy (Chatal et al., 2008). A number of radioimmunotherapy agents are
803 currently in development or in early phase trials, targeting other indications including
804 neuroblastoma (Kramer et al., 2007), leukaemia (Miederer et al., 2004) and ovarian
805 carcinoma (Andersson et al., 2009).

806 (82) So far no radioimmunotherapy agent has proved to be effective for solid cancers, or
807 has been approved by health authorities due to low tumour-to-normal tissue absorbed dose
808 ratios, although many agents have been investigated in clinical studies. Research continues to
809 enhance the efficacy of radioimmunotherapy by improving tumour-to-normal tissue ratios,
810 for example, using pre-targeting methods (Goldenberg et al., 2012), and by applying new
811 radionuclides including alpha emitters.

812 **2.8.1. Aim of treatment**

813 (83) As radioimmunotherapy encompasses a range of procedures, treatment aims are
814 largely dependent on the radiopharmaceutical and the treatment itself, although the aim of
815 treatment is generally to eradicate tumour tissues that express tumour-associated antigens.

816 **2.8.2. Treatment protocols**

817 (84) Treatment regimens vary widely for radioimmunotherapy procedures. ^{90}Y -
818 ibritumomab tiuxetan therapy has well-established treatment protocols. Rituximab at 250 mg
819 m^{-2} is infused over 4 hours, followed by an infusion per body weight of 14.8 MBq kg^{-1} of
820 ^{90}Y -ibritumomab tiuxetan, not exceeding 1,184 MBq. In some countries and regions, prior to
821 ^{90}Y -ibritumomab tiuxetan therapy, imaging with ^{111}In -ibritumomab tiuxetan is performed
822 according to a therapy protocol implemented to verify the expected biodistribution and

823 exclude patients who show an altered biodistribution, such as the rapid clearance from the
824 blood pool, with prominent liver, spleen, or marrow uptake (Hanaoka et al., 2015).

825 **2.8.3. Radiation dose to friends and family**

826 (85) Exposure of friends and family is dependent on the radionuclide administered and
827 the relevant procedures should be followed accordingly. Activity is excreted through body
828 fluids, primarily urine and perspiration. Care must therefore be taken when a patient is
829 discharged, and home circumstances should be taken into account.

830 **2.8.4. Radiation dose to staff and carers**

831 (86) Careful attention should be taken for handling of beta emitting radiopharmaceuticals
832 as similar to the previous section. Particularly attention should be taken to finger dose for
833 preparation of ⁹⁰Y- ibritumomab tiuxetan because high radiation dose has been reported
834 (ICRP, 2008; Vanhavere et al., 2012).

835 **2.8.5. Patient organ dosimetry**

836 (87) A large number of dosimetry studies have been performed related to
837 radioimmunotherapy procedures. In Phase III trials of ⁹⁰Y-ibritumomab tiuxetan, the median
838 estimated radiation absorbed doses were 0.71 and 14.84 Gy to the active bone marrow and
839 tumour, respectively (Wiseman et al., 2001). In radioimmunotherapy, radiation dose to
840 organs at risk including the liver, lung, intestine, and kidney in relation to given radiolabelled
841 antibodies should be evaluated carefully using clinical tests and imaging modalities to
842 prevent unexpected overdose delivery.

843 **2.8.6. Risks to patients**

844 (88) In cases of ¹³¹I- and ⁹⁰Y-ibritumomab radiolabelled antibodies, acute toxicity is
845 primarily haematologic, causing thrombocytopenia and leukocytopenia. This needs careful
846 management in patients with less bone marrow reserves due to prior repeated chemotherapies.
847 Immunogenic response against the antibody is also a potential concern and should be
848 monitored carefully. As with all therapy procedures, pregnancy/breastfeeding is a
849 contraindication, and patients should avoid conception.

850 **2.8.7. Recommendations**

851 (89) Individual absorbed dose estimates must be performed for treatment planning and
852 post administration verification of dosimetry on an individualised basis. Due to the range of
853 radionuclides used, this may in some cases entail the use of surrogate imaging agents (for
854 example ¹¹¹In in place of ⁹⁰Y).

855 **2.8.8. Emerging Technologies in Radioimmunotherapy**

856 (90) A number of new radiotherapeutics are currently under development, some of which
857 have already reached the stages of clinical studies to evaluate safety and efficacy in humans.
858 Examples of new methods that have lately attracted worldwide attention include, but are not
859 limited to, prostate-specific membrane antigen (PSMA) ligands for prostate cancer, and

860 radioimmunotherapy with alpha-emitters for haematological malignancies such as anti-CD33
861 antibody labelled with ^{213}Bi or ^{225}Ac for acute myeloid leukaemia. Another new approach to
862 radiopharmaceutical therapy involves pre-targeting techniques, which can enhance tumour-
863 to-normal tissue accumulation ratios, and therefore the anti-tumour effect of treatment. Pre-
864 targeting techniques, which are more complex than conventional techniques, might require
865 more tailored considerations in safe and efficacious usage. Radiological protection standards
866 should be established for these new methods although it will take some time until sufficient
867 data on radiation doses and risks, as well as on patient care, are accumulated in clinical
868 studies.

869 2.8.8.1. *Therapy with PSMA ligands*

870 (91) PSMA is overexpressed in prostate cancer, especially in de-differentiated or
871 castration-resistant cases. Radiolabelled ligands for radionuclide imaging aimed at PSMA
872 have recently been the subject of a number of studies showing high diagnostic accuracy in
873 detecting primary tumours, recurrence, and metastases with good detection rates. The intense
874 PSMA expression in prostate cancer also provides a promising approach to develop new
875 radiopharmaceuticals for therapy. Some PSMA ligands have advantages of high affinity that
876 produce good tumour-to-normal tissue contrast as well as the ability to be labelled with ^{68}Ga
877 for imaging and ^{177}Lu for therapy. Several studies have reported promising results for
878 response rates and a favourable safety profile after therapy with ^{177}Lu -PSMA-617 in patients
879 with metastatic castration-resistant prostate cancer (Rahbar et al., 2017). Another application
880 of PSMA ligands in radiopharmaceutical therapy has been reported as an initial experience
881 with targeted ^{225}Ac -PSMA-617 alpha-therapy in a limited number of patients (Kratochwil et
882 al., 2016). Such alpha-emitter-labelled PSMA ligands may have high potential for treatment
883 of prostate cancer.

884 2.8.8.2. *Radioimmunotherapy with alpha-emitters*

885 (92) Because alpha-particles have a short range and a high linear energy transfer,
886 radioimmunotherapy with alpha-emitters offers the potential for efficient tumour cell killing
887 while sparing surrounding normal cells (Jurcic and Rosenblat, 2014). To date, clinical studies
888 of alpha-particle immunotherapy for acute myeloid leukaemia (AML) have focused on the
889 myeloid cell surface antigen CD33 as a target using monoclonal antibodies. Clinical studies
890 demonstrated safety, feasibility, and anti-leukaemic effects of ^{213}Bi -labelled anti-CD33
891 antibodies. A next-generation compound containing ^{225}Ac , half-life of 10 days, was
892 developed because the use of ^{213}Bi is limited by its short half-life of 46 minutes (Jurcic and
893 Rosenblat, 2014).

894 2.8.8.3. *Pre-targeting techniques*

895 (93) For the enhancement of efficacy of radionuclide therapy as well as radionuclide
896 imaging, pre-targeting strategies have been introduced. An example of pre-targeting
897 techniques is an approach of radioimmunotherapy in which the antibody is not labelled but
898 used to provide binding sites to small molecular weight radioactivity vectors. Such
899 techniques have been shown to increase tumour to non-target uptake ratios and anti-tumour
900 efficacy has been demonstrated in clinical studies (Chatal et al., 1995; Kraeber-Bodere et al.,
901 2006). Another example of pre-targeting techniques involves affibody (small proteins

902 engineered to bind to a high number of target proteins) molecule-based peptide nucleic acid
903 (PNA)-mediated pre-targeting, which increased radionuclide uptake in tumours in preclinical
904 studies (Honarvar et al., 2016).

905 **2.9. Intra-arterial Treatment of Hepatocellular Carcinoma and Liver** 906 **Metastases (Selective Internal Radiation Therapy: SIRT)**

907 (94) Hepatocellular carcinoma (HCC) and liver metastases may be treated via direct
908 infusion of a radiotherapeutic substance into the hepatic artery, by selectively catheterising
909 the hepatic artery branches that supply the tumours. The underlying basis for this procedure is
910 that liver tumours preferentially take their blood supply from the hepatic artery while normal
911 liver is predominantly fed by the portal vein. In recent years two commercial products, both
912 radiolabelled with ^{90}Y , have become the mainstay for these treatments. Glass microspheres
913 (Therasphere® BTG, Ontario, Canada) and resin microspheres (SIR-Spheres®, SIRTex
914 Medical Limited Sydney, Australia) have similar properties, although differ in terms of the
915 size of the particles and the concentration of activity on each sphere (Giammarile et al., 2011).
916 ^{166}Ho -microspheres are also currently under development (Smits et al., 2012). The procedure
917 also involves initial angiography and embolisation of branches not supplying a tumour before
918 microspheres are injected.

919 (95) This treatment offers the potential to deliver high absorbed doses to small and large
920 liver lesions with precision targeting. Potential disadvantages include a relatively invasive
921 procedure and the possibility of irradiation of normal tissue (primarily lungs, gut and normal
922 liver) that can have fatal implications (Giammarile et al., 2011).

923 **2.9.1. Aim of treatment**

924 (96) The primary aim of treatment is palliative, although complete responses and long
925 remissions have been reported.

926 **2.9.2. Treatment protocols**

927 (97) A number of formulae are employed to determine the level of activity to administer.
928 Current treatment protocols for microspheres, including mono-compartmental and partition
929 models, are predominantly based on levels of activity administered or on activity per body
930 surface area. Lung shunting is considered the most serious risk. For this reason a pre-therapy
931 whole-body $^{99\text{m}}\text{Tc}$ -MAA (macro-aggregated albumin) scan is performed and administered
932 activities are modified accordingly. If the lung shunt is too great, ^{90}Y microsphere
933 administration is contraindicated. The potential for redistribution to bowel, stomach or
934 pancreas must also be considered (Lambert, 2010). Post therapy scanning is usually
935 performed of the liver to ensure uptake. ^{90}Y bremsstrahlung imaging is most commonly used,
936 although in recent years, PET imaging has been developed following successful investigation
937 into the low positron yield of ^{90}Y , which is sufficient for the high concentrations of activity
938 localised in tumour and normal liver (Lhommel et al., 2010).

939 (98) There are no standardised treatment protocols or guidelines for ^{131}I -lipiodol. This has
940 not been considered an option for treatment of children and young people due to the concerns
941 of protection (Giammarile et al., 2011).

942 2.9.3. Radiation dose to friends and family

943 (99) As full physical retention is assumed for the microsphere treatments and ^{90}Y is
944 primarily a beta emitter, less stringent radiation protection issues are required and should be
945 addressed according to national guidelines.

946 2.9.4. Radiation dose to staff and carers

947 (100) Although microspheres are not metabolised and are considered as medical devices,
948 they must be treated as unsealed sources of radiation and standard precautions must be taken.
949 Standard precautions should be taken for care and imaging. Treatment with ^{131}I -lipiodol must
950 be subject to the usual restrictions involving this radionuclide.

951 2.9.5. Patient organ dosimetry

952 (101) Dosimetry is performed to guide treatment in few centres. Methods based on
953 calculations of the absorbed doses delivered to tumours and to normal liver (partition or
954 multi-compartmental modelling) have been developed although there are as yet no published
955 standard methodologies (Cremonesi et al., 2014) and gross assumptions are frequently made.
956 For example, the dosimetry method developed for glass spheres is used to calculate the mean
957 absorbed doses to the whole liver, inclusive of any tumour involvement. In recent years post-
958 therapy imaging and dosimetry have been developed using the low positron emission from
959 ^{90}Y which enables the use of PET (Willowson et al., 2015).

960 2.9.6. Risks to patients

961 (102) Microspheres are considered as medical devices and are not subject to active uptake
962 in normal organs. Irradiation of normal liver parenchyma, either from localisation within the
963 liver or from cross irradiation from localisation in liver tumours, is always a risk factor that
964 must be considered as this may cause radiation hepatitis. Radiation induced liver disease has
965 not as yet been clearly defined. There is evidence that an initial state of cirrhosis affects the
966 tolerability to radioembolisation (Chiesa et al., 2011). Delivery of radiation to the pancreas
967 will cause abdominal pain, acute pancreatitis or peptic ulceration. Lung shunting occurs when
968 administered activity passes into the pulmonary circulation and may result in radiation
969 pneumonitis. Inadvertent delivery to the gall bladder may result in cholecystitis. Shunting to
970 lungs, the GI tract or pancreas will vary from one procedure to the next and absorbed dose
971 limiting toxicity is therefore not possible to predict without pre-therapy biodistribution
972 scanning. Treatment verification is essential following therapy administration as infusion
973 locations may not be guaranteed and indeed may be modified from the pre-therapy work up.
974 As with all therapy procedures, pregnancy/breastfeeding is a contraindication, and patients
975 should avoid conception.

976 2.9.7. Recommendations

977 (103) The potential to induce severe toxicity or even to cause death, combined with the
978 probability of undertreating many patients, necessitates the use of personalised dosimetry for
979 treatment planning. The lack of certainty regarding the ability of the pre-therapy $^{99\text{m}}\text{Tc}$ -MAA
980 imaging study to predict the absorbed dose distribution delivered at therapy, exacerbated by

981 the possibility of administering the therapy to a different location from that used for the tracer
982 study, render post-treatment verification essential if the effect of treatment is to be understood.

983 **2.10. Treatment of Arthritis (Radionuclide Synovectomy)**

984 (104) The administration of radiopharmaceuticals for the treatment of rheumatoid or
985 osteoarthritis has been used for over 40 years (Ansell et al., 1963) and has become well
986 established and widely used. It is also used for treatment of haemophilic synovitis. This is
987 considered to be a cost effective and well tolerated option with significant advantages over
988 surgery and intra-articular administrations of steroids or chemical synovectomy.

989 (105) Following initial administrations with ^{198}Au , radionuclides with higher beta-particle
990 energies and with longer path length are now commonly used, including ^{90}Y and ^{32}P colloid
991 for larger joints such as the knee, ^{186}Re -colloid for smaller joints including elbows and ankle,
992 and ^{169}Er -citrate for metatarsophalangeal joint (Knut, 2015).

993 **2.10.1. Aim of treatment**

994 (106) The aim of radiosynovectomy is to reduce swelling and to provide pain relief.
995 Reduction of knee joint swelling has been seen in over 40% of patients and pain relief in
996 88%. Wrist, elbow, shoulder, ankle and hip joints have shown significant improvement and
997 restoration of normal function and long-term pain relief has been achieved in about 70% of
998 small finger joints. In haemophilic arthropathies complete cessation of bleeding has been
999 seen in 60% of patients and improved mobility in 75% (Das, 2007).

1000 **2.10.2. Treatment protocols**

1001 (107) Radiopharmaceuticals for synovectomy can be administered at intervals, typically 3
1002 months apart, following a successful first treatment. Repeated treatments are more effective
1003 than single treatments with higher activity. Current levels of activity administered have little
1004 evidence base and are derived empirically (Johnson et al., 1995).

1005 **2.10.3. Radiation dose to friends and family**

1006 (108) Dose to friends and family are not likely to be higher than those from standard
1007 diagnostic examinations.

1008 **2.10.4. Radiation dose to staff and carers**

1009 (109) Procedures are standardised as for diagnostic administrations, and sensible
1010 precautions must be undertaken, with the use of syringe shields where necessary. Exposures
1011 of radiopharmacists and nurses were found to be within acceptable limits, although for the
1012 therapists working in centres with high number of patients, the effective dose was reported to
1013 be 21 μSv for six treatments (Lancelot et al., 2008).

1014 **2.10.5. Patient dosimetry**

1015 (110) Uncertainties in absorbed dose calculations were addressed almost 40 years ago
1016 (Bowring and Keeling, 1978) when it was considered that the challenges of uptake and target

1017 localisation, quantification of the activity and monitoring the retention were scientifically and
1018 logistically prohibitive. A comprehensive approach to dosimetry for radiosynovectomy
1019 ideally requires a Monte Carlo approach which enables the production of depth dose profiles
1020 for any given radionuclide (Johnson et al., 1995).

1021 **2.10.6. Risks to patients**

1022 (111) The limited range of intra-articular injected radionuclides, while *in situ*, ensures little
1023 irradiation of adjacent tissues. Reported side effects are rare and are generally related to the
1024 administration procedure (comprising joint inflammation and skin necrosis from extra
1025 articular administrations). The radiation exposure of the whole body of patients is very low
1026 because the limited range of the beta emissions (10 mm for ^{90}Y and up to 1 mm for ^{169}Er). No
1027 genotoxic effects were found in peripheral blood following administration of ^{90}Y -citrate in
1028 children with haemophilic synovitis (Klett et al., 1999; Turkmen et al., 2007). Absorbed
1029 doses delivered to lymph nodes, liver, spleen and whole-body have been calculated as 619
1030 (154-1644) mGy, 62 (15-165) mGy, 62 (15-165) mGy and 37 (9-99) mGy, and leakage rates
1031 from sequential imaging are reported to be $> 2\%$ (Klett et al., 1999). In cases of 48 h
1032 immobilisation after therapy, the leakage rate of radio-colloids is $> 2\%$ (Klett et al., 1999). A
1033 large Canadian study of patients receiving radiosynovectomy with ^{90}Y , no increase in the
1034 incidence of cancer was observed in a study of 2412 adult patients with a variety of
1035 underlying conditions although the study concluded that further investigation was needed for
1036 procedures for younger patients (Infante-Rivard et al., 2012). As with all therapy procedures,
1037 pregnancy/breastfeeding is a contraindication, and patients should avoid conception.

1038 **2.10.7. Recommendations**

1039 (112) Leakage of particulates has been demonstrated to be low in animal models with
1040 sequential gamma camera imaging and is expected to be low in humans (Noble et al., 1983).
1041 However, studies are needed to confirm the assumption.
1042

1043

1044

3. BIOKINETIC DATA COLLECTION

- 1045 • **In radiopharmaceutical therapy, the absorbed dose delivered to an organ or tissue**
1046 **is governed by the radiopharmaceutical uptake into and clearance from the source**
1047 **organ and surrounding organs, combined with the radionuclide physical half-life.**
1048 **Biokinetic data can be collected using techniques that vary in complexity. These**
1049 **should be chosen with regard to the accuracy required for the particular task.**
- 1050 • **Acquisition should follow protocols (or Standard Operating Procedures) to assure**
1051 **consistency and allow for comparisons.**

1052 **3.1. Whole-body Activity**

1053 (113) Although radionuclides for therapy need to have short range emissions to focus dose
1054 delivery within target tissues, whole-body monitoring of organ/tissue uptake and retention
1055 rely on the radionuclide also having penetrating photon emissions. For radionuclides having
1056 penetrating photon or bremsstrahlung emissions, the activity in the whole-body can be
1057 measured most easily and accurately with a detector at a distance larger than 2 m. The first
1058 data point is taken before the patient micturates so that this value can be used for normalizing
1059 the data set to 100%. All subsequent measurements must be performed in the same geometry.
1060 This procedure is correct only if the sensitivity of the probe is independent of the distribution
1061 of activity in the patient. This is normally the case, if the photons scattered by the patient are
1062 eliminated by spectroscopic measurements including only the photo-peak of the radionuclide
1063 in question (Lassmann et al., 2008).

1064 (114) The determination of activity of the whole-body can alternatively be performed by
1065 repeated whole-body scans with a gamma camera. Post-therapeutically it has to be
1066 ascertained that the dead time correction of the camera is set up properly (Delpon et al., 2002;
1067 Hänscheid et al., 2006; Lassmann et al., 2008).

1068 **3.2. Activity in the Blood**

1069 (115) This method is typically applied for determining the absorbed dose to the blood
1070 (Lassmann et al., 2008; Hänscheid et al., 2009) or to the bone marrow (Hindorf et al., 2010).
1071 The kinetics of activity in blood is typically measured by serial sampling of heparinised blood
1072 and subsequent measurement in a calibrated well counter. In particular, dependent on the
1073 biokinetics of the compound considered, at least one blood sample needs to be withdrawn at a
1074 later stage (> 96 h) (Lassmann et al., 2008).

1075 **3.3. Organ and Tumour Activity**

1076 **3.3.1. Quantitative imaging**

1077 (116) Quantitatively accurate imaging is required for treatment planning and evaluation of
1078 radiopharmaceutical therapy. Over the past years there has been a great deal of progress in

1079 the development of methods for accurately quantifying nuclear medicine images. However,
1080 propagation of these methods into clinics has been slow.

1081 (117) Achieving quantification requires appropriate equipment, software and human
1082 resources. The level of these requirements depends on the imaging task. For example,
1083 quantifying activity in a tumour in the lungs requires more sophisticated resources than
1084 quantifying whole-body activity. However, detailed knowledge about the requisite levels of
1085 resources is not widely available or appreciated.

1086 (118) While, in general, multiple use of sophisticated imaging devices provide for better
1087 determination of the biokinetics of a radiopharmaceutical, this benefit must be weighed
1088 against what is practically achievable. On the one hand, a few probe measurements could
1089 provide valuable insights into whole-body retention in the individual patient. On the other
1090 hand, multiple SPECT/CT or PET/CT sessions might be needed for initial evaluation of
1091 efficiency and toxicity of novel therapeutic radiopharmaceuticals.

1092 (119) The type and number of imaging sessions needed for a particular patient undergoing
1093 radiopharmaceutical treatment must thus be optimised. Consideration should include what
1094 personnel and equipment are available; the financial and logistical hurdles for using them; the
1095 expected accuracy of the quantification; any radiation protection concerns involved in the
1096 imaging sessions; and any possible patient discomfort.

1097 (120) This section provides a brief overview of the technology involved in quantitatively
1098 accurate imaging. More thorough descriptions such as the IAEA Human Health Reports No.
1099 9 (Quantitative Nuclear Medicine: Concepts, Requirements and Methods) can be consulted
1100 for more details (IAEA, 2014b).

1101 3.3.2. Planar imaging

1102 (121) Today, planar imaging with a gamma camera for dosimetric purposes is useful for
1103 determining organ uptake and clearance biokinetics, and individual organ overlap must be
1104 accurately assessed, taking into account attenuation, scatter, and background correction
1105 (Siegel et al., 1999).

1106 (122) Planar images are most commonly used with dual-head cameras (Siegel et al., 1999;
1107 Glatting et al., 2005). For opposite heads the pixel-wise geometric mean is a first-order
1108 approximation for the activity in the corresponding pixel (conjugate view method). The
1109 dependency of the measured count-rate I_{PQ} [counts s^{-1}] of the activity A_{PQ} [MBq] of a point
1110 source PQ is

$$1111 \quad I_{PQ} = C \cdot A_{PQ} \cdot e^{-\mu_e x} \quad (3.1)$$

1112 where C is the calibration coefficient [counts $MBq^{-1} s^{-1}$] of the camera head, μ_e [$1 cm^{-1}$] is the
1113 effective linear attenuation coefficient and x [cm] the depth of the point source in the body.
1114 The geometric mean of the count rates G [counts s^{-1}] for two opposite camera heads and the
1115 thickness of the body D [cm] is calculated as

$$1116 \quad G = \sqrt{I_a \cdot I_p} = A_{PQ} \cdot C \cdot \sqrt{e^{-\mu_e x} \cdot e^{-\mu_e (D-x)}} = A_{PQ} \cdot C \cdot e^{-\mu_e D/2} \quad (3.2)$$

1117
1118
1119
1120
1121
1122

1123 where I_a and I_p are the measured anterior and posterior count rates and $C = \sqrt{C_a \cdot C_p}$ the
 1124 calibration factor for the geometric man of both camera heads. Solving eq. (3.2) for the
 1125 unknown activity A_{PQ} results in

1126

$$1127 \quad A_{PQ} = \frac{\sqrt{I_a \cdot I_p}}{C} e^{\mu_c D/2} \quad (3.3)$$

1128
 1129 (123) Thus, the thickness of the investigated object or patient and linear attenuation
 1130 coefficient are required for determining the activity of a point source when using two
 1131 opposite camera heads. This equation is only valid if the sensitivity of the camera head is not
 1132 dependent on the distance from the source. As this is only approximately true, the error can
 1133 be more than 100%, depending on the nuclide, the energy window and the collimator in
 1134 comparison to the mid-position of the point source (Glating and Lassmann, 2007).

1135 **3.3.3. SPECT/CT**

1136 (124) The market share of SPECT/CT systems, i.e. gamma cameras, which are coupled
 1137 with a CT for attenuation correction, has grown in recent years. Today, to measure activity in
 1138 the accumulating organs and tumours using imaging techniques, quantification by means of
 1139 SPECT/CT for at least one data point is state-of-the-art. Due to the inclusion of scattering and
 1140 attenuation correction, accuracies of better than 10% are achievable in phantom
 1141 measurements (Dewaraja et al., 2012, 2013).

1142 (125) The calibration of imaging systems is essential for patient-specific dosimetry in
 1143 nuclear medicine therapy. Unfortunately, there is no universal calibration standardisation
 1144 method published for the gamma cameras and radionuclides used in radiopharmaceutical
 1145 therapy today. In addition, large calibration sources for nuclides which either are used pre-
 1146 therapeutically as a substitute for ^{90}Y (^{111}In) or therapeutically used nuclides are not available
 1147 (^{131}I , ^{177}Lu). Therefore the calibration relies on ‘in-house’ produced calibration phantoms,
 1148 which are filled with the appropriate radionuclide solutions.

1149 (126) For the calibration and for determining the optimal parameters for quantifying
 1150 SPECT/CT a large calibration source in air and in water filled with the radioactive substances
 1151 in question should be scanned and reconstructed, to obtain the appropriate values. For the
 1152 best quantification, the following conditions should be met (Dewaraja et al., 2012, 2013;
 1153 Fernández et al, 2012; Zimmerman et al., 2016):

- 1154 - A finer angular grid with reduced scanning times is better than a course grid (Dewaraja et
 1155 al., 2012).
- 1156 - MIRD Pamphlet 26 (Ljungberg et al., 2016) states that iterative methods require a certain
 1157 number of updates before reaching an acceptable image quality. MIRD Pamphlet 23
 1158 (Dewaraja et al., 2012) defines the convergence as when the 90% recovery has been
 1159 reached, this is a level of ‘high reconstruction accuracy’. A general ‘rule-of-thumb’ is
 1160 that more complex reconstruction problems (where more corrections are included in the
 1161 algorithm) require a larger number of iterations to reach convergence. It is important to
 1162 investigate this dependency and optimise reconstruction parameters using data from
 1163 phantom studies and simulations but also sample patient data with representative activity
 1164 distributions and counting statistics. Due to the limited spatial resolution of SPECT/CT it
 1165 is advisable when using the CT volume or a fixed threshold for volume-of-interest

1166 drawing to implement corrections for the partial-volume effect. For an empirical
1167 correction of the spill-out of the counts the volume-of-interest may be increased to
1168 account for the spatial resolution of the SPECT/CT system in comparison to the volume
1169 measured by CT.

1170 - For ^{111}In and ^{177}Lu there is no difference in accuracy whether one or two photopeaks are
1171 chosen, provided that the energy windows for the photopeak and the adjacent scatter
1172 windows are chosen correctly. For ^{177}Lu , however, care has to be taken that, for an
1173 incorrect window setting of the scatter window for the 113 keV peak, the quantification
1174 might show a larger error than 10% (Ljungberg et al., 2016).

1175 (127) In principle, the required organ volumes can be obtained from tomographic emission
1176 measurements. The accuracy of these methods, however, especially in smaller structures, is
1177 limited due to their relatively poor spatial resolution. In addition, motion artefacts can mask
1178 the true organ volume. Therefore, it seems useful to use high-resolution anatomical
1179 procedures such as CT scans or MRI for the determination of volumes.

1180 **3.3.4. PET/CT**

1181 (128) The role of PET/CT for therapeutic radiopharmaceuticals has mostly focused on
1182 using positron-emitting surrogates of the therapeutic radionuclides, such as ^{124}I for ^{131}I , and
1183 ^{86}Y for ^{90}Y treatments.

1184 (129) The possibility of quantitative PET/CT imaging of ^{90}Y has, however, been
1185 demonstrated for SIRT (Carlier et al., 2015). A multicentre comparison of quantitative ^{90}Y
1186 PET/CT for dosimetric purposes after radioembolisation with resin microspheres showed that
1187 the current generation time-of-flight scanners can consistently reconstruct ^{90}Y activity
1188 concentrations, but they underestimate activity concentrations in small structures (≤ 37 mm
1189 diameter) within a warm background due to partial volume effects and constraints of the
1190 reconstruction algorithm (Willowson et al., 2015).

1191 **3.4. Quantitative Protocols**

1192 **3.4.1. Quantitative Imaging Protocols**

1193 (130) Protocols (or Standard Operating Procedures) ensure consistency of data acquisition
1194 and processing. A protocol (or a set of protocols) should describe all the steps required to
1195 obtain satisfactory clinical data and measurements from them. A protocol should be written
1196 for any quantitative imaging task.

1197 (131) The expertise required for designing protocols differs from that required to
1198 implement them and different personnel may be required. Typically, the protocol could be
1199 written by a trained medical physicist and the medical staff, while technologists can be
1200 trained to execute protocols.

1201 (132) Quality assurance and quality control tasks (QA/QC) should be performed at a
1202 specified frequency to ensure that the equipment is operating as intended. The schedule for
1203 QA/QC procedures should be specified in the protocol. QA/QC results should be
1204 systematically provided along with all the data related to the protocol.

1205 **3.4.2. Pharmacokinetics and the Integration of the Time-activity-curve**

1206 (133) The choice of acquisition times for determining the uptake and retention of activity
1207 in an organ or structure of interest determines the reliability of the assessment of the number
1208 of decays in this organ/structure (Glatting and Lassmann, 2007). This value is calculated by
1209 integrating the respective time-activity curves. According to the MIRD Pamphlet 21
1210 nomenclatures (Bolch et al., 2009) this quantity is called time-integrated activity in the source
1211 region (old term: ‘cumulated activity’). The number of data points needed depends upon the
1212 biokinetics in the respective organ/tissue. As a rule of thumb, one needs at least three
1213 measurements for correctly fitting each of the exponential functions required for describing
1214 the biokinetics (Siegel et al., 1999). The determination of the number of exponential
1215 functions for an adequate description of the biokinetics is not trivial, as, in principle, for an
1216 exact representation an infinite number of functions are necessary. The number of functions
1217 used in reality depends strongly on the tolerated errors of the fitting process.

1218 (134) For the integration of the time activity curves and calculation of the time-integrated
1219 activity coefficient, a software solution presented by Kletting et al. (2013) offers a range of
1220 possible functions by means of statistical criteria.

1221 (135) As the number of scans in patients is limited MIRD Pamphlet 16 (Siegel et al., 1999)
1222 recommends five measurements at $T_e/3$, $2T_e/3$, $3T_e/2$, $3T_e$, $5T_e$; T_e is the effective half-life in
1223 the organ/structure considered. However, in practice, the choice will depend also on the
1224 availability of the equipment and the clinical condition of the patient.

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4. METHODS FOR ABSORBED DOSE CALCULATIONS

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- **The use of radiopharmaceuticals for cancer treatment requires detailed, patient-specific dosimetry and dose planning for assessments of absorbed dose to both normal tissues and tumours based on the quantitative measurements of organ activity over time, and organ mass.**

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(136) The use of radiopharmaceuticals for cancer treatment requires detailed, patient-specific dosimetry for assessments of absorbed dose to normal tissues and to tumour tissues. In therapy treatment planning, the calculation of radiation absorbed dose to internal organs, tissues, and the whole-body, is a fundamentally important aspect of successfully achieving clinical objectives. Since radiopharmaceuticals are usually administered systemically or orally, radionuclide therapy necessarily involves delivery of some radiation energy to normal organs and tissues. The amount of activity administered should be great enough to effectively treat the neoplasm while minimizing radiation dose to normal tissues. Therefore, the activity that may be safely administered can be determined by assessing the maximum absorbed doses to the most important, toxicity-limiting normal tissues.

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(137) Quantitative measurements of organ activity over time, and organ mass, are essential to calculate absorbed doses. In radiopharmaceutical therapy treatment planning and for patient safety, it is usually more important to accurately assess normal organ dose than to assess tumour dose. The ratio of the target region (or tumour) dose to the limiting normal organ dose, or $D_{\text{tumour}}/D_{\text{normal}}$, is the therapeutic index. Therapeutic index provides an estimate of therapeutic efficacy and safety.

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4.1. Purpose for Absorbed Dose Calculations

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(138) Absorbed dose calculations are performed prior to therapy on the basis of measurements made following a trace-labelled diagnostic infusion, or post-therapy on the basis of measurements following a therapy infusion. Internal radiation dosimetry serves several fundamental purposes in radiopharmaceutical therapy and radiation protection, including:

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- To evaluate the safety and efficacy of a therapeutic agent;
- To provide an information source for discussing anticipated radiation doses with patients;
- To plan an appropriate treatment for radiopharmaceutical therapy;
- To predict short-term and long-term radiation effects or dose-related biological endpoints associated with radiotherapy, and to correlate biological effects with radiation dose;
- To provide a required list of estimated radiation doses to internal organs from radiopharmaceuticals;
- To fulfil legal obligations and demonstrate regulatory compliance;
- To serve as a component of complete patient medical records.

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4.2. Data for Absorbed Dose Calculations

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(139) In radiopharmaceutical therapy, the time of intake and the amount of activity administered represent known or established quantities, determined by prescription, based on

1266 prior estimates of the radiation dose that will be needed to achieve beneficial therapy
1267 outcomes.

1268 (140) The major challenge in radiation dose assessment is to assess accurately the time-
1269 course of retention of radionuclide in organs and tumour tissue. The pharmacokinetic
1270 behaviour of radiolabelled drug products is analysed and determined by direct measurements
1271 (nuclear medicine imaging), direct bioassay (blood and excreta counting), and/or tissue
1272 biopsy counting (see Chapter 3). Direct measurements may be supplemented by
1273 pharmacokinetic modelling using population parametric values. For therapy treatment
1274 planning or post-infusion follow-up, individual patient measurements are more reliable than
1275 estimates based on population biokinetic models. Since the biodistribution and metabolic
1276 behaviour of radiopharmaceuticals usually vary from one patient to another, patient-specific
1277 measurements are needed to determine patient-specific biokinetic parameters.

1278 (141) Direct measurements of organ or tissue radioactivity must account for the geometry
1279 and density of the source organ or tissue, organ size and mass, potential overlap, thickness of
1280 tissue between the organ and the detector, and the spatial distribution of activity within a
1281 tissue. Measurements are corrected for body and detector background, detector dead time,
1282 and photon attenuation and scatter that may influence the accuracy of direct counting.

1283 (142) For any radionuclide, the information needed to calculate absorbed dose includes:
1284 the total activity administered to the patient and time of infusion, the fraction of the
1285 administered activity that is taken up by each major source organ or tissue, and the time-
1286 dependent retention and clearance of activity in each major source organ through complete
1287 radiological decay.

1288 (143) In the medical setting, measurements of organ activity may be made using calibrated
1289 nuclear medicine systems; these include planar gamma camera (anterior/posterior) imaging,
1290 single-photon emission computed tomography (SPECT) imaging, positron emission
1291 tomography (PET), and single crystal (sodium iodide or other scintillator) photon detectors.
1292 The patient is placed within the field of view for quantitative imaging over thoracic or
1293 abdominal regions; alternatively, the patient may receive a whole-body scan for region-of-
1294 interest measurements. The imaging procedure is repeated at pre-determined time points
1295 following a base-line (pre-injection) count and a post-injection image immediately after
1296 radiopharmaceutical infusion (near time zero). Markers are used to correctly position the
1297 patient for repetitive measurements. The technician selects regions of interest by outlining the
1298 major organs or tissue regions. In addition to all regions of interest, it is important to measure
1299 whole-body radioactivity over time.

1300 (144) Instrument counts in selected regions of interest are converted to units of activity
1301 (Bq) using radionuclide standards, patient-thickness measurements, background subtraction,
1302 attenuation correction, and scatter correction techniques. Such instrument counts require
1303 availability of photon emissions for quantitative counting. When it is not possible to
1304 determine precise activity concentrations in organs and tissues with time, estimates may be
1305 made using biokinetic or pharmacokinetic modelling. The quality of the assessment depends
1306 on the validity of the model parameters assumed. Modelling can provide important
1307 information where data are lacking, but the models are rarely patient-specific, and potential
1308 errors that are introduced must be taken into account.

1309 **4.3. Absorbed Dose**

1310 (145) Absorbed dose is the fundamental radiation quantity that describes energy deposition
 1311 by ionising radiation in an absorbing medium (ICRU, 2016); absorbed dose applies to all
 1312 radiation exposures, all types of ionising radiation, any absorbing medium, and all biological
 1313 targets and geometries. Calculation of absorbed dose from intake of radionuclides requires
 1314 information about the amount of radioactivity present over time periods through complete
 1315 decay, the mass and geometry of the target tissue, and all physical factors governing energy
 1316 deposition after radionuclide decay (ICRP, 2015a, 2015b).

1317 (146) In radiopharmaceutical therapy, the time of intake and the amount of activity
 1318 administered represent known or established quantities. The amounts of radioactivity present
 1319 in organs and tissues after administration may be determined by direct quantitative imaging
 1320 or by sample measurement and pharmacokinetic modelling. Methods that have been
 1321 developed for medical internal radiation dosimetry greatly simplify the dose-assessment task
 1322 without compromising on essential details. Nuclear medicine imaging, image rendering, and
 1323 computational capabilities are evolving to meet the needs for accurate and reliable internal
 1324 dosimetry. Current methods extend from the whole-organ to the cellular and multi-cellular
 1325 levels, and may be applied to either uniform or non-uniform radionuclide distributions within
 1326 organs and tissues. Patient-specific methods are preferred over generic model assumptions.

1327 (147) For radionuclide therapy, the relevant dosimetric quantity associated with immediate
 1328 deterministic effects in radiopharmaceutical therapy is the absorbed dose, in units of J kg⁻¹.
 1329 The absorbed dose, *D*, to an organ or tissue is the energy imparted, ϵ , per unit mass of tissue,
 1330 *m*, from all ionising radiation components that contribute energy to the target tissue mass.

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1332
$$D = \epsilon/m \quad \text{Gy (J kg}^{-1}\text{)} \quad (4.1)$$

1333

1334 (148) When applied to radionuclides administered to a living biological system, where the
 1335 source region is the same as the target region, the general absorbed dose equation includes a
 1336 biological retention function to account for radionuclide metabolism and clearance, as well as
 1337 the fraction of energy that is captured or absorbed in the target region.

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$$D = \left(\frac{AEY\phi}{m} \right) \int_0^t B(t) dt \quad \text{Gy (J kg}^{-1}\text{)} \quad (4.2)$$

1340

1341 where *D* is the mean absorbed dose, *A* is the activity of the radionuclide (Bq), *EY* is the total
 1342 energy emitted (joule) by activity in the organ or tissue (product of the particle energy and
 1343 yield), ϕ is the fraction of that energy that is absorbed, *m* is the mass of the target region (kg),
 1344 and $\int_0^t B(t) dt$ is the biological retention of the activity integrated from time *t* = 0 (injection)
 1345 through complete decay (*t* = ∞), or for any specific time period, *t* (seconds or hours). The
 1346 mass of the target organ should be determined from medical imaging; but standard model
 1347 values for organ mass may be used if precise data are not available. Equation (4.2) rearranged
 1348 is:

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1350
$$D = A \int_0^t B(t) dt \left(\frac{EY\phi}{m} \right) \quad \text{Gy (J kg}^{-1}\text{)} \quad (4.3)$$

1351

1352 (149) The patient comprises multiple source and target organs or tissues. The radiation
 1353 absorbed dose to any organ or tissue includes all energy deposition event contributions from
 1354 (a) radioactivity contained within the organ (the self-organ dose), and from (b) all energy
 1355 depositions originating from radioactivity contained in all other organs and tissues of the
 1356 whole body (the cross-organ dose). The mean absorbed dose is calculated by accounting for
 1357 the physical half-life, biological retention, all radioactive emissions by a given radionuclide,
 1358 and the individual absorbed fractions for all radioactive emissions from that radionuclide for
 1359 any specified source-target geometry in the human body. The complex geometries
 1360 represented by the human body for any age, sex, height, weight, variations in organ size, and
 1361 differences in tissue density (skeleton, soft tissue, lungs), taken together, present formidable
 1362 challenges for a comprehensive calculation that can account for all important determinants of
 1363 ϵ/m for any specified target region. The dose calculation must account for differences in
 1364 radionuclide biokinetics (uptake, retention, and clearance) unique to each organ or tissue for
 1365 the radiopharmaceutical of interest, together with factors that may determine unique
 1366 metabolic rates and health status of individual patients and which render differences in
 1367 pharmacokinetics from one patient to another.

1368 (150) The medical internal radiation dose (MIRD) schema (Loevinger and Berman, 1968)
 1369 was developed to account for all physical, biological, and geometric factors for all energy
 1370 contributions to absorbed dose for any target tissue from radionuclides in multiple source
 1371 organs and remainder tissues. Since 1968, the MIRD schema has evolved to accommodate
 1372 modern anatomical views by CT or MRI, voxel-level activity distributions, Monte-Carlo
 1373 energy transport codes, pharmacokinetic compartment models, and radiobiological response
 1374 parameters.

1375 (151) After administration of a radiopharmaceutical via intravenous injection, the drug
 1376 product redistributes quickly throughout the organs and tissues of the body, and all organs
 1377 and tissues receive some amount of radiation dose. However, by definition in the MIRD
 1378 schema, the source organ or region, r_S , is defined as any tissue mass, organ, tumour, or the
 1379 whole body for which data are available to determine a time-activity curve. The target organ
 1380 or region r_T , is defined as any organ or tissue for which an absorbed dose can be calculated.

1381 (152) Using the updated MIRD/ICRP formalism and nomenclature (Bolch et al., 2009;
 1382 ICRP, 2015b), the mean absorbed dose $D(r_T, \tau)$ to a target tissue r_T over a defined dose-
 1383 integration period τ (infinity for short-lived radionuclides) following administration of a
 1384 radioactive material to the medical patient is:

1385
 1386
$$D(r_T, \tau) = \sum_{r_S} \int_0^\tau A(r_S, t) S(r_T \leftarrow r_S, t) dt \quad \text{Gy (J kg}^{-1}\text{)} \quad (4.4)$$

1387
 1388 where the quantity $S(r_T \leftarrow r_S, t)$ is the radionuclide-specific quantity representing the mean
 1389 absorbed dose rate to target region r_T at time t after administration, per activity present in
 1390 source region r_S (Snyder et al., 1969; Bolch et al., 2009). For a specific radionuclide and for a
 1391 well-defined geometry representing the source-target pair,
 1392

1393
$$S(r_T \leftarrow r_S, t) = \frac{1}{m(r_T, t)} \sum_i E_i Y_i \phi(r_T \leftarrow r_S, E_i, t) = \frac{1}{m(r_T, t)} \sum_i \Delta_i \phi(r_T \leftarrow r_S, E_i, t) \quad (4.5)$$

1394
 1395 where E_i and Y_i are the energy and yield (number per nuclear transition), respectively, of each
 1396 radiation particle or photon i emitted by the radionuclide; Δ_i is their product (or mean energy
 1397 emitted per nuclear transition); and the quantity $\phi(r_T \leftarrow r_S, E_i, t)$ is the absorbed fraction of

1398 radiation energy E_i emitted by the source region r_s at time t that is absorbed in the target
 1399 tissue r_T .

1400 (153) If the quantity $A(r_s, t)$ is normalised to a unit administered activity A_0 and is
 1401 designated as the quantity $a(r_s, t)$, then the absorbed dose coefficient $d(r_T, \tau)$ in target tissue
 1402 r_T is (Bolch et al., 2009):

$$1403 \quad d(r_T, \tau) = \sum_{r_s} \int_0^\tau a(r_s, t) S(r_T \leftarrow r_s, t) dt \quad \text{Gy Bq}^{-1} \quad (4.6)$$

1404
 1405 where $a(r_s, t) = A(r_s, t)/A_0$ is the fraction of the administered radioactivity remaining in the
 1406 source tissue r_s at any time t post-infusion. The fraction $a(r_s, t)$ is the quantity that is
 1407 measured for radiation dosimetry in the patient by region-of-interest quantitative imaging
 1408 using clinical nuclear medicine instruments.

1409 (154) Equation (4.4) may be simplified, when time dependence of S is neglected, using the
 1410 time-independent expression:

$$1411 \quad D(r_T, \tau) = \sum_{r_s} \tilde{A}(r_s, \tau) S(r_T \leftarrow r_s) \text{ Gy} \quad (4.7)$$

1412
 1413 where the quantity $\tilde{A}(r_s, \tau)$ represents the time-integrated activity (or total number of nuclear
 1414 decay transitions) in source region r_s for the dose-integration period τ , and where:

$$1415 \quad \tilde{A}(r_s, \tau) = \int_0^\tau A(r_s, t) dt \quad \text{Bq s} \quad (4.8)$$

1416
 1417 (155) Fully implemented, the MIRD/ICRP formalism represented by equation (4.7)
 1418 accounts for all source regions, all target organs, respectively all source-target geometries,
 1419 and all radioactive emissions contributing to absorbed dose. Tabulated values of S have been
 1420 published to simplify internal dose calculations for simple source-target geometries. For all
 1421 other cases, the specific absorbed fractions for a radionuclide and computational phantom-
 1422 model must be calculated individually using a Monte Carlo nuclear transport code that
 1423 accounts for geometry, tissue compositions, and absorber densities. Dosimetry calculations
 1424 may be performed with a number of commercially available software packages or software
 1425 developed in-house (Guy et al., 2003; McKay, 2003; Glatting et al., 2005; Stabin et al., 2005).
 1426 Software used for calculation of organ doses and effective doses by ICRP is available
 1427 (Andersson et al., 2014; ICRP, 2015a; www.idac-dose.org).

1431 4.4. Time-integrated Activity Coefficient in a Source Region

1432 (156) The time-integrated activity coefficient $\tilde{a}(r_s, \tau)$ is the area under the time-activity
 1433 curve representing the integral quantity $\int_0^\tau a(r_s, t) dt$ in equation (4.6). This quantity was
 1434 previously known as the residence time in earlier MIRD publications; it is equal to the ratio
 1435 of the time-integrated activity and the total administered activity, A_0 :

$$1436 \quad \tilde{a}(r_s, \tau) = \int_0^\tau a(r_s, t) dt = \tilde{A}(r_s, \tau) / A_0 \quad \text{Bq s Bq}^{-1}, \text{ or s} \quad (4.9)$$

1437
 1438 (157) The time-integrated activity coefficient is a common input value for software
 1439 programmes that implement the MIRD/ICRP schema for absorbed dose calculations. The
 1440 time-integrated activity coefficient for a source region may be determined by plotting the
 1441

1442 fraction of administered activity in that source region over time and evaluating the area under
1443 the curve. Several measurement data points, depending on the form of the mathematical
1444 function, are needed to establish a time-activity curve best represented by the plotted data
1445 (Siegel et al., 1999).

1446 (158) The counts obtained in an organ or tissue region of interest must be converted to
1447 units of radioactivity using appropriate measurement methods and calibration standards,
1448 including: daily quality assurance, patient positioning, patient-thickness measurements,
1449 background subtraction, attenuation correction, and scatter correction. In planar imaging, the
1450 geometric mean of counts obtained from anterior and posterior views is determined. The
1451 fraction of administered activity measured in the source region may be plotted as a function
1452 of time post-infusion. A mathematical function or time-activity curve should then be fitted to
1453 the plotted data using linear least-squares regression analysis. Physical decay is exponential,
1454 and biological uptake and clearance usually follow exponential patterns; therefore, an
1455 exponential function with one or more terms is usually an appropriate function to represent
1456 the plotted data. The fitted function is integrated numerically or analytically to yield the time-
1457 integrated activity coefficient.

1458 (159) Alternatively, the time-integrated activity coefficient for a source region may be
1459 calculated using dynamic modelling if the pharmacokinetic parameters associated with model
1460 compartments (source regions) and their associated transfer coefficients are known or can be
1461 determined iteratively. When combined with dosimetry subroutines, and following the
1462 general MIRD/ICRP schema, biokinetic models may also be used to calculate radiation
1463 absorbed doses to target regions directly.

1464 **4.5. Uncertainties in Absorbed Dose Calculations**

1465 (160) Uncertainty analyses provides information about the sources of bias (accuracy) and
1466 random variation (precision), respectively, and their magnitudes, that show the reliability and
1467 quality of absorbed dose calculations. Internal dose calculations involve many different
1468 measurements, complex anatomical geometries, and highly variable biological factors when
1469 applied to administered radiopharmaceuticals. Uncertainty propagation is therefore
1470 challenging and perhaps intractable when all details of measurements and sources of
1471 modelling errors must be accounted for. Nonetheless, the major sources of uncertainty should
1472 be recognised, acknowledged, and minimised as much as possible, to improve confidence in
1473 the estimated absorbed dose.

1474 (161) The total uncertainty in an estimate of the mean absorbed dose to an organ or tissue
1475 from a therapeutic radiopharmaceutical administered to a patient reflects different sources of
1476 uncertainty: (a) measurement uncertainties associated with quantitative imaging methods
1477 used to determine absolute activities in major source regions, (b) uncertainties in estimating
1478 integrated activity in organs/tissues and (c) the application of mathematical phantoms or
1479 standard reference models used to represent the anatomical organ geometries of live subjects.

1480 (162) With modern activity measurement instruments (“dose calibrators”), administered
1481 activities may be known to accuracies within a few percent. Differences between planned and
1482 actual administered activity are only minor contributors to the total uncertainty if regular
1483 quality control is performed (IAEA, 2006a). Uncertainties associated with variances in
1484 assumed mass of the target organ may be minimised with use of patient CT and 3-
1485 dimensional volumetric reconstructions.

1486 (163) Variations in estimated time-integrated activities for major source organs arise from
 1487 inherent difficulties in measuring and quantifying organ uptake, retention, and redistribution
 1488 of the radiopharmaceutical in tissues (Norrgrén et al., 2003; Jönsson et al., 2005).
 1489 Uncertainties associated with the shape of the time-activity curve may be minimised by
 1490 obtaining sufficient data points to establish the time-activity function and optimise statistical
 1491 fitting to the data. The most important data points are the initial organ uptake near time zero
 1492 after administration or completion of the infusion, and the last time point that weighs heavily
 1493 toward helping one to determine the slope of the long-term retention. Typically, a minimum
 1494 of four or five data points are needed at properly spaced collection times to minimise
 1495 uncertainty associated with area-under-curve analyses.

1496 (164) Variations in estimates of photon cross-organ contributions to a source region dose,
 1497 dependent on assumed distances between the source and target organs, contribute to
 1498 uncertainties in tabulated *S* values. Physical data, such as the radionuclide emission energies
 1499 and yields applicable to absorbed fraction calculations for target organs are well characterised
 1500 and do not contribute significantly to overall uncertainty.

1501 (165) Experimental measurement validation of calculated absorbed doses using reference
 1502 anthropomorphic phantoms and mathematical models have indicated agreement within 20 to
 1503 60%, depending on the degree to which subjects compare with the body size and shape
 1504 assumed in the calculations (Roedler, 1980).

1505 **4.6. Biologically Effective Dose (BED)**

1506 (166) When an absorbed dose is delivered by low-LET radiation at a low absorbed-dose
 1507 rate, the radiobiological effects are known to decrease as compared to those obtained for the
 1508 same absorbed dose delivered with a high dose rate. The decrease is associated with repair of
 1509 DNA damage during irradiation, and depends on the tissue repair capacity and the rate of
 1510 repair in relation to the time of radiation delivery. There are also other time-dependent factors
 1511 that may modify the cellular response, such as proliferation (repopulation), redistribution in
 1512 the cell cycle, and reoxygenation (Joiner and van der Kogel, 2015).

1513 (167) In radiopharmaceutical therapy the absorbed-dose rate in an organ or tissue is
 1514 governed by the radiopharmaceutical uptake and retention in the organ itself and surrounding
 1515 organs, combined with the radionuclide physical half-life. The radiation delivery can extend
 1516 over long times (days or even weeks) (Gleisner et al., 2015), the absorbed-dose rate varies
 1517 over time, and the mean absorbed-dose rate is considerably lower than in most other forms of
 1518 radiotherapy. There are also spatial heterogeneities, governed mainly by the molecular
 1519 mechanisms for radiopharmaceutical accumulation and the range of the particles that are
 1520 emitted at radioactive decay.

1521 (168) Applications of the linear-quadratic (LQ) radiobiological model were early
 1522 described for radiopharmaceutical therapy (Millar, 1991; Howell et al., 1994; Dale, 1996) to
 1523 estimate the fraction of cell surviving the irradiation, *SF*, as

$$1524 \quad SF = e^{-(\alpha D + G(T) \beta D^2)} \quad (4.10)$$

1525
 1526
 1527 (169) where *D* is the absorbed dose delivered from the start of irradiation until time *T*, and
 1528 α and β are radiobiological parameters that characterise the shape of the cell survival curve.
 1529 The first term in the exponent, linear in *D*, dominates the cell-survival curve at low absorbed
 1530 doses and has been interpreted to be associated with lethal DNA damage induced by single-

1531 particle tracks (Dale, 1996). The second, quadratic, term describes the increasingly
 1532 downward curvature for SF at higher absorbed doses and has been interpreted as effects from
 1533 pairwise interaction of sub-lethal lesions induced by two particle tracks. The function G ,
 1534 called the Lea-Catcheside factor, acts as a damping of the second term, and is deduced from
 1535 the perspective that there is a probability that the first sub-lethal DNA lesion is repaired
 1536 before the second is induced. G is formally defined as (Lea and Catcheside, 1942; Kellerer
 1537 and Rossi, 1974).

$$1539 \quad G(T) = \frac{2}{D^2} \int_0^T R(t) \left[\int_0^t R(w) \varphi(t-w) dw \right] dt \quad (4.11)$$

1540
 1541 where $R(t)$ is the absorbed-dose rate as function of time. The function $\varphi(t)$ describes the
 1542 loss of sub-lethal lesions due to repair and is often assumed to be a single-phase process with
 1543 a repair half time, T_{rep} , and rate constant $\mu = \ln(2) / T_{\text{rep}}$, such that

$$1544 \quad \varphi(t) = e^{-\mu t} \quad (4.12)$$

1545
 1546 although multi-phase repair processes have also been reported (Joiner and van der Kogel,
 1547 2009). The function $G(T)$ takes values between zero and one depending on the rate of repair
 1548 in relation to the rate of cell-lesion induction, in turn proportional to the absorbed-dose rate.

1549 (170) For most radionuclide therapies, irradiation continues until the radionuclide has
 1550 decayed or has been excreted. For an absorbed-dose rate function described by an effective
 1551 decay constant, λ , combined with equation (4.12) and integration in equation (4.11) to
 1552 infinity, $G(T)$ takes the form

$$1553 \quad \lim_{T \rightarrow \infty} G(T) = \frac{\lambda}{\lambda + \mu} \quad (4.13)$$

1554
 1555 Analytic solution of equation (4.11) for more complicated absorbed-dose rate patterns or
 1556 repair functions can become quite cumbersome. It was noted that the integral within brackets
 1557 in equation (4.10) can be described as a convolution (Gustafsson et al., 2013a). This
 1558 formulation allows for numerical implementation, which opens for application of more
 1559 complex absorbed-dose rate functions and repair functions other than mono-exponential
 1560 functions (Gustafsson et al., 2013b).

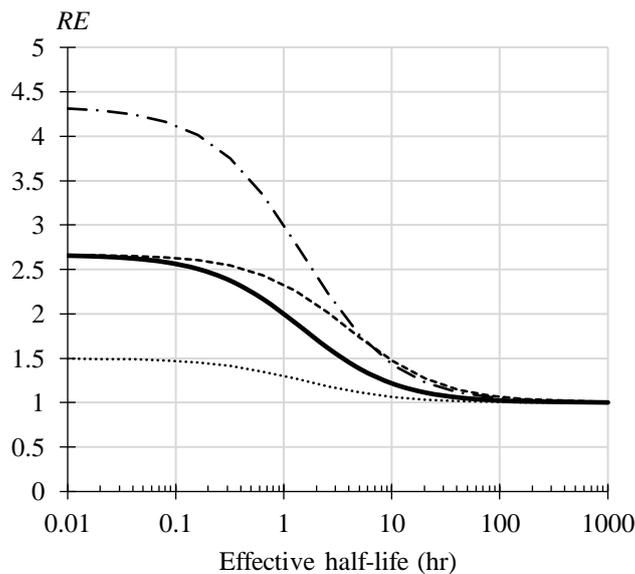
1561 (171) The biologically effective dose (BED) is a concept within the framework of the LQ
 1562 model (Barendsen, 1982; Fowler, 1989; Dale, 1996; Joiner and van der Kogel, 2009). It relies
 1563 on the idea of equieffective treatments, i.e. treatments that produce the same probability of
 1564 inducing a specific clinical (biological) endpoint (Bentzen et al., 2012). The main use of BED
 1565 is in external-beam radiotherapy and brachytherapy where it is a clinically accepted method
 1566 for conversion between different fractionation schemes and absorbed-dose rate patterns. In
 1567 radiopharmaceutical therapy its usefulness for describing clinically observed effects has been
 1568 demonstrated (Barone et al., 2005; Wessels et al., 2008; Strigari et al., 2010). Barone et al.
 1569 (2005) found that kidney toxicity correlated better to BED than to absorbed dose, and in
 1570 MIRD Pamphlet No. 20 (Wessels et al, 2008) these and other data were combined to find that
 1571 the relationship between BED and the incidence of renal complications was comparable to
 1572 that obtained for external-beam radiotherapy. Strigari et al. (2010) described a relationship
 1573 between BED and the normal tissue complication probability of liver.

1576 (172) For organs and tissue, the biologic effect is described in a functional form that is
 1577 equivalent to the logarithm of the cell killing in equation (4.10), i.e. $-\ln(S)$. The BED is then
 1578 calculated according to
 1579

$$BED = D + \frac{G(T)}{\alpha/\beta} D^2 = D \left(1 + \frac{G(T) \cdot D}{\alpha/\beta} \right) = D \cdot RE \quad (4.14)$$

1581 where the α/β value is characteristic for the organ or tissue and the endpoint, i.e. an observed
 1582 effect. The formulation of BED as the product of D and the relative effectiveness, RE , has
 1583 been given by Barendsen (1982) and Dale (1996). In this notation RE is the ratio of absorbed
 1584 doses required to yield a given equieffect, where the BED is the absorbed dose when given at
 1585 infinitesimally small fraction doses or infinitesimally low dose rate. The BED is higher or
 1586 equal to D , so RE is larger than, or equal to unity.
 1587

1588 (173) Figure 4.1 shows the value of RE for selected values of the different parameters in
 1589 equations (4.14) and (4.13). For short effective half-lives, G approaches unity and RE goes
 1590 towards a value valid for an instant delivery of the absorbed dose. For long effective half-
 1591 lives, G becomes small and RE approaches unity. Changes in D or α/β both result in
 1592 variations of RE along the vertical axis, whereas changes in the repair half-life induce shifts
 1593 along the horizontal direction.
 1594



1595 Fig. 4.1. The relative effectiveness, RE , obtained from equations (4.14) and (4.13). As baseline values,
 1596 shown by the solid line, parameters used are $D=5$ Gy, $\alpha/\beta=3$ Gy, and $T_{rep} = 1.5$ h. The dash-dotted
 1597 line is obtained when the absorbed dose is changed to 10 Gy, and dotted line when α/β is changed to
 1598 10 Gy. The dashed line is obtained when the repair half-life is changed to 4 hr.
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5. SPECIFIC RADIATION PROTECTION ISSUES

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- **The need for guidance on radiation protection for people at risk of exposure include hospital staff, members of the patient’s family including children, and carers, neighbours, visitors to the household, co-workers, those encountered in public places, on public transport or at public events, and the general public. These risks can be effectively managed and mitigated with well-trained staff, appropriate facilities, and the use of patient-specific radiation safety precaution instructions.**
- **Special consideration should be given to pregnant women exposed to ionising radiation. Pregnancy is a contraindication to radiopharmaceutical therapy, unless the therapy is life-saving. Breastfeeding should be discontinued in radiopharmaceutical therapy patients.**
- **Accident prevention in radiation therapy should be an integral part of the design of facilities, equipment, and administration procedures.**
- **Optimisation of staff exposures include consideration of education and training, equipment design, proper shielding and handling of sources, personal protective equipment and tools as well as awareness and engagement in radiation protection.**
- **Individual monitoring of the whole body and extremities must be considered for staff during the management of radiopharmaceutical therapy patients and in the preparation and administration of radiopharmaceuticals.**
- **Radiation sources used in radiopharmaceutical therapy can contribute significant doses to medical personnel and others who may spend time within or adjacent to rooms that contain such sources. Meaningful dose reduction and contamination control can be achieved through the use of appropriate procedures, and facility and room design, including shielding where appropriate.**
- **Medical practitioners should provide all necessary medical care consistent with patient safety and appropriate medical management. Radiation protection considerations should not prevent or delay life-saving operations in the event that surgery is required. Staff should be informed when a patient may pose a radioactive hazard, and advice and training should be provided prior to administrations.**
- **The decision to hospitalise or release a patient after therapy should be made on an individual basis considering factors such as the residual activity in the patient, the patient’s wishes, family considerations (particularly the presence of children), environmental factors, and existing guidance and regulations. Advice on specific radiation protection precautions should be provided to patients and carers.**

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5.1. Introduction

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(174) The use of radiation for radiopharmaceutical therapy is a planned exposure situation – it needs to be under regulatory control, with an appropriate authorisation in place from the regulatory body before operation can commence (ICRP, 2007a). Misadministration, spills and other such incidents or accidents can give rise to potential exposure, but these remain part of the planned exposure situation as their occurrence is considered in the granting of an authorisation (Carlsson and LeHeron, 2014). Each of the categories of exposure of individuals (medical, occupational, and public) need to be considered in radiopharmaceutical therapy. In addition, the three fundamental principles of radiological protection (justification,

1644 optimisation, and limitation) (ICRP, 2007a) are applicable. In a nuclear medicine facility,
1645 occupational and public exposures are subject to all three principles, whereas medical
1646 exposure of patients is subject to the first two only (ICRP, 2007b).

1647 (175) Implementation of radiological protection for radiopharmaceutical therapy is an
1648 essential part of the system for implementing quality medical practice in a facility. The most
1649 important aspect is to establish a safety culture among staff, such that protection and accident
1650 prevention are regarded as inherent to daily activities. Several guidelines for implementation
1651 of radiation protection in a nuclear medicine facility have been developed (IAEA, 2005a;
1652 2005b, 2009, 2014; Sisson et al 2011) that address: programme elements, responsibilities,
1653 education and training, facility design, monitoring, waste, and health surveillance. These
1654 should be consulted as applicable in addition to the considerations given in subsequent
1655 sections of this publication.

1656 **5.2. Requirements for Radiopharmaceutical Therapy Treatment Rooms and Wards**

1657 (176) The following aims should be considered in the design of radiopharmaceutical
1658 therapy treatment rooms and wards: optimising protection to reduce the exposure to external
1659 radiation and contamination, maintaining low radiation background levels to avoid
1660 interference with imaging equipment, meeting pharmaceutical requirements, sequestering
1661 waste appropriately, and ensuring safety and security of sources (locks and controlled access).

1662 (177) Typically, rooms for high-activity patients should have separate toilet and washing
1663 facilities. The design of safe and comfortable accommodation for visitors is important. Floors
1664 and other surfaces should be covered with smooth, continuous, non-absorbent, and non-
1665 porous surfaces that can be easily cleaned and decontaminated. The walls should be finished
1666 in a smooth and washable surface, for example, painted with washable, non-porous paint.
1667 Secure areas should be provided with bins for the temporary storage of linen and waste
1668 contaminated with radioactivity.

1669 (178) Proper shielding and ventilation is required for storage of bulk radioiodine
1670 containers. Preparation of activity for administration of radioiodine should be performed in
1671 hoods with adequate airflow to protect staff and extraction systems capable of adsorbing
1672 contaminants prior to emission. Adequate containment and exhaust should be provided for
1673 the storage of radioiodine waste and articles with residual contamination.

1674 (179) Radiopharmaceutical therapy patients in unshielded hospital rooms may expose
1675 persons in adjacent areas to levels of radiation that might cause dose constraints to be
1676 exceeded. Vacating adjacent rooms or areas or installing shielding (e.g. permanent poured
1677 concrete, solid concrete block, steel plates, lead sheets or portable shielding devices) may be
1678 necessary to ensure dose constraints are maintained in adjacent areas (Chu et al., 2016).
1679 Areas on floors immediately above and below such patient's rooms as well as on the same
1680 floor must be considered. Table 5.1 gives typical shielding effectiveness values for ^{131}I which
1681 requires the most intensive shielding. Exposure or dose rates should be measured after each
1682 radiopharmaceutical administration, or worst-case scenario evaluations documented to
1683 confirm that these are below levels that could cause a dose constraint to be exceeded.
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Table 5.1. Typical shielding effectiveness values for ¹³¹I.

	Half value layer	Tenth value layer
Lead (Delacroix et al, 2002)	3.0 mm	11 mm
Concrete (Schleien et al, 1998)	5.5 mm	18 mm

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(180) A monitoring system should be established in facilities, considering protection of the public and staff. For permanent shielding evaluations, it is important to properly design structural shielding, considering anticipated dose rates in controlled and supervised areas (IAEA, 2006b). Dose rates in occupied areas adjacent to the radionuclide treatment room should be monitored and results recorded to ensure that dose constraints are not exceeded and protection is optimised.

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(181) It is preferable that patient treatment rooms be for individual patients and adjacent to each other. If this is not possible, appropriate shielding is required between the treated patient and a neighbouring patient. When required, shielding should be provided for nurses and visitors of radiopharmaceutical therapy patients; movable shields may be used within patient rooms. When required, prior to each treatment, movable shields should be placed close to the patient's bed in such a way that exposure of the nurses caring for the patient is minimised. This is achieved by anticipating the nurse's tasks, positions and movements throughout the room.

1704 **5.3. Patients (Medical Exposure)**

1705 **5.3.1. Justification and optimisation of protection**

(182) In radiation therapy, the aim is to eradicate neoplastic (or otherwise diseased) target tissue or to palliate the patient's symptoms. Some deterministic damage (tissue reactions) to surrounding tissue and some risk of stochastic effects in exposed non-target tissues are inevitable, but the goal of all radiation therapy is to optimise the relationship between the probability of tumour control and normal tissue complications. If the dose to the target tissue is too low, the therapy will be ineffective and the exposures will not have been justified (ICRP, 2007b). However, the protection of tissues outside the target volume is an integral part of dose planning. Thus, the principle of optimisation of protection is applied to nuclear medicine therapy procedures that have been justified with an emphasis that the appropriate radiopharmaceutical and activity are selected, correctly calculated, measured and administered so that the activity is primarily localised in the organ(s) of interest, while the activity in the rest of the body is maintained ALARA (ICRP, 2001b).

1718 **5.3.2. Considerations prior to therapy**

(183) A risk assessment must be performed prior to radiopharmaceutical therapy to ensure that the patient is self-caring, able to tolerate isolation (if appropriate), and able to comply with radiation precautions (when necessary).

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1722 5.3.3. Pregnancy

1723 (184) Pregnancy is a contraindication to radiopharmaceutical therapy, unless the therapy is
1724 life-saving. This advice is all the more valid for radioiodine therapy and for other
1725 radionuclides with the potential to accumulate in fetal tissues. Beyond 10-13 weeks of
1726 gestation, the foetal thyroid may receive extremely high doses in cases of therapy using ¹³¹I-
1727 iodide (Watson et al., 1989; Berg et al., 1998; ICRP, 2008). The possibility of pregnancy
1728 should be carefully excluded before administration. Therefore, where treatment is likely or
1729 anticipated, the patient should also be advised to take appropriate contraceptive measures in
1730 the time prior to therapy.

1731 (185) Before any procedure using ionising radiation, it is important to determine whether a
1732 female patient is pregnant with a blood pregnancy test performed before time (usually within
1733 72 hours) of treatment in all women, from menarche to 2 years after menopause, who could
1734 become pregnant. There may be exceptions to the requirement for a pregnancy test, but there
1735 must be incontrovertible evidence that pregnancy is impossible, for example, surgical
1736 hysterectomy (Sisson et al, 2011).

1737 (186) The feasibility and performance of medical exposures during pregnancy require
1738 specific consideration owing to the radiation sensitivity of the developing embryo/foetus
1739 (ICRP, 2001a, 2007a). The ICRP has given detailed guidance in *Publications 84* (ICRP,
1740 2000) and *105* (ICRP, 2007b). Radiation risks after prenatal radiation exposure are discussed
1741 in detail in ICRP *Publication 90* (ICRP, 2003).

1742 (187) A major problem occurs when a female, who is not thought to be pregnant, is treated
1743 for thyroid carcinoma and is found to be pregnant after the administration of radioiodine. If a
1744 patient is discovered to be pregnant shortly after a therapeutic radio-iodine administration,
1745 maternal hydration and frequent voiding should be encouraged to help eliminate maternal
1746 radioactivity and to reduce radioiodine residence time in the bladder. If the pregnancy is
1747 discovered within several hours of the radioiodine administration and the fetus is old enough
1748 to have a functional thyroid, one should consider thyroid-blocking using potassium iodide. If
1749 the pregnancy is discovered later, the placental transfer of radioiodine can result in very high
1750 absorbed doses to the fetal thyroid that may cause significant damage. Since the fetal whole-
1751 body dose is usually below 100 mGy, there is no reason to terminate the pregnancy (ICRP,
1752 2000); however, the mother should be given usual levels of replacement thyroid hormone.

1753 5.3.4. Breastfeeding

1754 (188) Female patients should be advised that breastfeeding is absolutely contraindicated
1755 after therapeutic administration of radionuclides. Any therapeutic radiopharmaceutical
1756 administered orally, intravenously or arterially is potentially hazardous to the child, and
1757 breast feeding must cease. Intracavitary administrations of suspended particles such as
1758 yttrium-90 silicate represent little hazard; however, it would still be wise to cease feeding.
1759 Breastfeeding should be discontinued in radiopharmaceutical therapy patients for two
1760 reasons. The first and most critical is to prevent radionuclides in milk from reaching the
1761 infant (and in particular the infant's thyroid gland in radioiodine therapies) (Azizi and Smyth,
1762 2009) in addition to the external radiation from the patient to the infant. The second reason is
1763 to limit radiation of the breast tissue, which may concentrate certain radionuclides during
1764 lactation. The restriction period depends on the radionuclide administered for therapy. In case
1765 of ¹³¹I treatment, the patient should stop breastfeeding 6 weeks before the treatment (Sisson et
1766 al., 2011) and should not resume it after the treatment for her current child.

1767 **5.3.5. Radioactive patients on dialysis**

1768 (189) The care of patients receiving radiopharmaceutical therapy and who are on dialysis
 1769 may require additional consideration and radiation protection/medical physics experts should
 1770 be consulted. In general, for systemic treatments, these patients will not biologically clear
 1771 radioactive materials in the same manner as typical patients since the clearance is highly
 1772 dependent on the schedule of dialysis sessions.

1773 **5.3.6. Conception**

1774 (190) Conception should be avoided in both males and females, with clear advice from 4-
 1775 12 months following radiopharmaceutical therapy. Table 5.2 obtained from *Publication 106*
 1776 (ICRP, 2004) gives additional information on precaution times for female avoidance of
 1777 conception for specific radionuclide therapies. Pregnancy should also be delayed based on the
 1778 need to normalise hormonal responses (e.g. in the case of thyroid therapy) for a successful
 1779 pregnancy and healthy infant development, and to ensure that additional radiation treatment
 1780 is not imminent (Sisson et al., 2011).

1781 (191) It is widely recommended, on the basis of prudence, that male patients take steps to
 1782 avoid fathering children during the months immediately following therapy. However, there is
 1783 no strong evidence base to support this view (Sawka et al., 2008a, 2008b).

1784 Table 5.2. Periods for avoiding pregnancy after radiopharmaceutical therapy to ensure that the dose to
 1785 the fetus will not exceed 1 mGy*
 1786
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Radionuclide and form	For treatment of:	All activities up to: (MBq)	Avoid pregnancy (months)
¹³¹ I-iodide	Hyperthyroidism	800	4
¹³¹ I-iodide	Thyroid cancer	6,000	4
¹³¹ I-mIBG	Neuroendocrine tumours	7,500	3
³² P-phosphate	Myeloproliferative disease	200	3
⁸⁹ Sr-chloride	Bone metastases	150	24
⁹⁰ Y-colloid	Arthritic joints	400	0
⁹⁰ Y-colloid	Malignancies	4,000	1

1788 * Selected data from Table 13.3 of ICRP *Publication 94* (ICRP, 2004).
 1789

1790 **5.3.7. Prevention of medical errors with radiopharmaceuticals**

1791 (192) Accident prevention in radiation therapy should be an integral part of the design of
 1792 equipment and premises and of the working procedures (ICRP, 2007b). A key feature of
 1793 accident prevention has long been the use of multiple safeguards against the consequences of
 1794 failures through design of equipment and facilities as well as the use of working procedures.
 1795 Working procedures should require key decisions, especially in radiation therapy, to be
 1796 subject to independent confirmation. Effective communication between all the staff and the
 1797 patient is a vital part of the process. Remedial actions in emergency situations associated with

1798 the use of radioactive materials in therapy need to be identified prior to any programme
1799 launch (e.g. the dose from an excessive or erroneous administration of radioiodine in therapy
1800 may be reduced by the early administration of stable iodine as potassium iodide or iodate to
1801 reduce the uptake of radioiodine by the thyroid).

1802 (193) Care should be exercised in avoiding administration of a therapeutic
1803 radiopharmaceutical to the wrong patient. In addition, prior to administration, the following
1804 should be verified to match the prescription:

- 1805 - Identification of the patient by two independent means;
- 1806 - Identity of the radionuclide;
- 1807 - Identity of the radiopharmaceutical;
- 1808 - Total activity;
- 1809 - Date and time of administration;
- 1810 - Patients have been given information about their own safety.

1811 (194) Records of the therapeutic radiopharmaceutical, data from dose planning,
1812 administered activity, the date and time of administration, and verification of the initial and
1813 residual assay should be entered in some form in the patient's medical record (ICRP 2007b)
1814 together with the activity at the time of discharge. It should be maintained at the hospital and
1815 given to the patient along with written precautionary instructions.

1816 **5.4. Staff (Occupational Exposure)**

1817 (195) Exposure of workers may arise from unsealed sources either through external
1818 irradiation of the body or through entry of radioactive substances into the body. The
1819 principles for the protection of workers from ionising radiation, including those in medicine,
1820 are discussed in *Publication 75* (ICRP, 1997) and in *Publication 103* (ICRP, 2007a).
1821 Generally, the yearly effective dose to staff working full time in nuclear medicine with
1822 optimised protection should be well below 5 mSv. Besides facility and equipment design,
1823 proper shielding and handling of sources as well as personal protective equipment and tools
1824 are important in such optimisation (ICRP, 2008; Carlsson and LeHeron, 2014). Optimisation
1825 is also achieved through education and training (ICRP, 2009), resulting in awareness and
1826 engagement in radiological protection. Detailed requirements for protection against
1827 occupational exposure for nuclear medicine facilities are given in several documents (ICRP,
1828 2007a, 2007b; IAEA, 2011, 2014a) and recommendations on how to meet these requirements
1829 are given in IAEA Safety Guides (IAEA, 1999) and in particular IAEA Safety Reports Series
1830 No. 40 (IAEA, 2005a).

1831 (196) Pregnant women and persons under the age of 18 y should not be involved in
1832 procedures with therapeutic levels of radiopharmaceuticals.

1833 **5.4.1. Protective equipment and tools**

1834 (197) Protective clothing should be used in radiopharmaceutical therapy areas where there
1835 is a likelihood of contamination. The clothing serves both to protect the body of the wearer
1836 and to help to prevent the transfer of contamination to other areas. Protective clothing should
1837 be removed prior to going to other areas such as staff rooms. The protective clothing may
1838 include laboratory gowns, waterproof gloves, overshoes ('booties'), and caps and masks for
1839 aseptic work. Radiation safety glasses should be worn to protect the eyes from beta radiation
1840 and contamination of the eye. When beta emitters are handled, two layers of gloves should be

1841 worn to avoid contamination of the skin. There should be emphasis on use of shielding, tools
1842 and work practices that minimise exposure by preventing direct handling of vials, syringes
1843 and contaminated articles.

1844 (198) In radiopharmaceutical therapy, most of the occupational exposures come from ^{131}I ,
1845 which emits 364-keV photons. The attenuation by a lead apron at this energy is minimal (less
1846 than a factor of two) and is unlikely to result in significant dose reduction and may not justify
1847 the additional weight and discomfort of wearing such protective equipment. Typically,
1848 thicker permanent or mobile lead shielding may be more effectively applied for those
1849 situations that warrant its use. Radiation protection experts/medical physicists should
1850 determine the need and types of shielding required for each situation. The use of automatic
1851 injection systems will significantly reduce the absorbed dose to the staff members (Rushforth
1852 et al., 2017).

1853 (199) Administration is normally by the oral route, intravenous injection (systemic), intra-
1854 articular injection or instillation of colloidal suspensions into closed body cavities
1855 (intracavitary). Shielded syringes should be utilised during the intravenous administration of
1856 radiopharmaceuticals as necessary to ensure that extremity doses are maintained below
1857 occupational dose constraints. Absorbent materials or pads should be placed underneath an
1858 injection or infusion site. The facility Radiation Protection Officer (RPO) should be consulted
1859 to determine the necessity of other protective equipment (e.g. shoe covers etc.) for particular
1860 radiopharmaceutical therapies.

1861 (200) For oral administrations of therapeutic radiopharmaceuticals, the radioactive
1862 material should be placed in a shielded, spill-proof container. Care should be taken to
1863 minimise the chance for splashing liquid or for dropping capsules. Appropriate long-handled
1864 tools should be utilised when handling unshielded radioactive materials. For intravenous
1865 administrations by bolus injections, when dose rates warrant, the syringe should be placed
1866 within a syringe shield (plastic for beta emitting radionuclides to minimise bremsstrahlung,
1867 high Z materials for photon-emitting radionuclides) with a transparent window to allow for
1868 visualisation of the material in the syringe. For intravenous administrations by slower drip or
1869 infusions, the activity container should be placed within a suitable shield. For high-energy
1870 photons, a significant thickness of lead or other high-Z material may need to be evaluated. In
1871 addition, consideration should be given for shielding pumps and lines.

1872 (201) Procedures for administering a therapeutic radiopharmaceutical shall include
1873 considerations to ensure as complete a delivery as possible of the prescribed therapeutic
1874 activity. Any residual activity in syringes, tubing, filters or other equipment utilised for
1875 administration should be assayed. Where appropriate, equipment should be flushed or rinsed
1876 with isotonic saline (or another physiological buffer) for parenteral administration or water
1877 for oral administrations. All materials utilised in administrations shall be considered as
1878 medical and radioactive waste, and should be labelled with the radionuclide, a radiation
1879 precaution sticker, and stored and or disposed of in a manner consistent with local regulations.

1880 **5.4.2. Individual monitoring**

1881 (202) Regular individual monitoring of external exposure should be performed during the
1882 management of radiopharmaceutical therapy patients and in the preparation and
1883 administration of radiopharmaceuticals. Extremity monitoring should also be carried out for
1884 handling of radiopharmaceuticals taking into account the potential differences between
1885 exposure of the dosimeter and the location of the extremity where the highest dose is likely to
1886 be received (Rimpler et al., 2011; Sans-Merce et al., 2011).

1887 (203) Significant doses to the hands can be received during the administration of
1888 radionuclides which emit high-energy beta-radiation. If adequate protection measures are not
1889 in place, the exposure of the fingers will be high, and doses of many tens and even hundreds
1890 of mSv have been reported from single patient administrations for a number of different ⁹⁰Y
1891 therapies (Barth and Mielcarek, 2002; Liepe et al., 2005a; Rimpler et al., 2007; Rimpler and
1892 Barth, 2007). The use of grasp forceps to hold the needle significantly reduces the dose to the
1893 hands (ICRP, 2008). Training and educational materials are provided by ICRP
1894 (<http://www.icrp.org/page.asp?id=35>) and other organisations ([http://www.oramed-](http://www.oramed-fp7.eu/en/Training%20material)
1895 [fp7.eu/en/Training%20material](http://www.oramed-fp7.eu/en/Training%20material)).

1896 (204) Staff to be monitored in a nuclear medicine facility should include all those who
1897 work routinely with radionuclides or nursing or other staff who spend time with therapy
1898 patients. Monitoring for internal contamination is rarely necessary in general nuclear
1899 medicine procedures on radiological protection grounds, but it may be useful in providing
1900 reassurance to staff (Carlsson and LeHeron, 2014). The circumstances in which internal
1901 monitoring becomes advisable are those where staff use significant quantities of ¹³¹I for
1902 therapy. These staff should be included in a programme of regular thyroid uptake
1903 measurements.

1904 **5.4.3. Contamination control procedures**

1905 (205) In the event of a large-volume spill of radiopharmaceuticals, blood, urine or vomitus,
1906 medical practitioners or staff should cover the spill with an absorbent material and
1907 immediately contact the radiation protection/medical physics experts for appropriate clean-up
1908 assistance and specific instructions. After such a spillage, the following actions should be
1909 taken:

- 1910 - The RPO should immediately be informed and directly supervise the clean-up;
- 1911 - Absorbent pads should be placed over the spill to prevent further spread of
1912 contamination;
- 1913 - All people not involved in the spill should leave the area immediately;
- 1914 - Access to the contaminated area should be restricted;
- 1915 - All people involved in the spill should be monitored for contamination when leaving the
1916 room;
- 1917 - If clothing is contaminated, it should be removed and placed in a plastic bag labelled
1918 'radioactive';
- 1919 - If contamination of skin occurs, the area should be washed immediately;
- 1920 - If contamination of an eye occurs, it should be flushed with large quantities of water.

1921 (206) Upon discharge and release of the patient, all remaining waste and contaminated
1922 items should be removed and segregated into bags for disposable items and launderable items.

1923 **5.4.4. Surveys and monitoring**

1924 (207) For area monitoring, the operational quantity for assessing effective dose is the
1925 ambient dose equivalent, $H^*(10)$ (ICRU, 1993; ICRP, 1996b, 2010). The ambient dose
1926 equivalent rate from the patient should be determined. This information will assist in deriving
1927 appropriate arrangements for entry by visitors and staff and for patient release. Rooms with
1928 radiotherapy patients should be controlled areas.

1929 **5.4.5. Emergency patient care**

1930 (208) Medical practitioners should provide all necessary medical care consistent with
1931 patient safety and appropriate medical management. Unless otherwise specified by the
1932 facility RPO, nurses, physicians and other health care personnel are to perform all routine
1933 duties, including those requiring direct patient contact, in a normal manner.

1934 (209) Ward nurses should be informed when a patient may pose a radioactive hazard, and
1935 advice and training should be provided regularly.

1936 (210) Radiation protection considerations should not prevent or delay life-saving
1937 operations in the event that surgery is required. The following precautions should be
1938 observed:

- 1939 - The operating room staff should be notified;
- 1940 - Operating procedures should be modified under the supervision of the RPO to minimise
1941 exposure and the spread of contamination;
- 1942 - Protective equipment may be used as long as efficiency and speed are not affected;
- 1943 - Rotation of personnel may be necessary if the surgical procedure is lengthy;
- 1944 - The RPO should monitor all individuals involved;
- 1945 - Doses to members of staff should be measured as required.

1946 (211) If the medical condition of a patient deteriorates such that intensive nursing care
1947 becomes necessary, such care is a priority and should not be delayed. However, the advice of
1948 the RPO should be sought immediately. In the event of deterioration in the patient's medical
1949 condition, frequent or continual monitoring of the patient may be necessary (e.g. septic shock,
1950 pulmonary oedema, stroke or myocardial infarction).

1951 (212) Life-saving efforts shall take precedence over consideration of radiation exposures
1952 received by medical personnel. This is particularly important for therapy patients containing
1953 large amounts of radionuclides. Medical personnel should, therefore, proceed with
1954 emergency care (e.g. when a patient has suffered a stroke), while taking precautions against
1955 the spread of contamination and minimising external exposure. The staff should avoid direct
1956 contact with the patient's mouth, and all members of the emergency team should wear
1957 protective gloves. Medical staff should be informed and trained on how to deal with
1958 radioactive patients. Rehearsals of the procedures should be held periodically.

1959 **5.4.6. Transfer of patients to another healthcare facility**

1960 (213) Some patients may need to be transferred to another healthcare facility (i.e. another
1961 hospital, skilled nursing facility, nursing home or hospice, etc.) following therapy treatments.
1962 In such a case, care must be taken that, in addition to practical measures and advice to ensure
1963 safety of other staff, compliance with any legal requirements relevant to the second institution
1964 is assured (IAEA, 2009) Patients transferred to another healthcare facility should meet the
1965 criteria for unrestricted clearance. However, the possibility for the generation of low-level
1966 radioactive waste should be examined by the RPO of the treating facility and any issues
1967 should be discussed with the facility accepting the patient transfer. In the rare event that a
1968 patient being transferred to another healthcare facility does not meet the criteria for
1969 unrestricted clearance, the RPO shall ensure that the facility accepting the patient transfer has
1970 an appropriate registration or licence that would allow acceptance of the patient with
1971 therapeutic amounts of radioactive materials on board. The RPO should provide radiation
1972 safety information and precautions, if any, for the patient and for the receiving healthcare
1973 facility.

1974 **5.4.7. Death of the patient following radiopharmaceutical therapy**

1975 (214) In the event that a patient dies within the treating healthcare facility while still
1976 containing a therapeutic quantity of radioactive material, the treating medical practitioner and
1977 the RPO shall be notified immediately.

1978 (215) In cases where the death occurs in a hospital, access to the room occupied by the
1979 deceased should be controlled until the room has been decontaminated and surveyed.
1980 Radioactive bodies should be identified as potential hazards by a specified form of identifier.
1981 Identification of the possibility that a body may contain radioactive substances relies on
1982 information provided in the patient records, the information card or information gleaned from
1983 relatives or others. A body bag may need to be used to contain leakage of radioactive
1984 substances. To minimise external radiation, the body may need to be retained in a controlled
1985 area.

1986 (216) The dose constraints applying to pathology staff responsible for the conduct of
1987 autopsy examinations will be either those for the general public or those for radiation workers,
1988 depending on the training and classification of the staff concerned. These constraints and the
1989 radiation safety procedures to be applied in practice should be determined in close
1990 consultation with the RPO from the department in which the therapy was administered.

1991 (217) Unsealed radioactive substances may be present in a particular body cavity or organ,
1992 or they may have concentrated after systemic administration (e.g. ^{131}I in the thyroid gland).
1993 Drainage of the cavity or excision of the organ will reduce exposure if undertaken at the start
1994 of the autopsy. In addition, care should be given with respect to organs with significant
1995 activity. In cases where the patient had received a dose of beta-emitting colloid or spheres
1996 (e.g. ^{32}P chromic phosphate into a body cavity or ^{90}Y microspheres into the liver), significant
1997 activity may be present in the cavity fluid or in the embolised organ. Beta radiation sources
1998 may provide significant dose to the hands because they will be in close contact with body
1999 tissues and fluids (NCRP, 2006). Autopsy and pathology staff should wear standard
2000 protective clothing (i.e. gloves, lab coats, eye protection, etc.) and personnel monitoring
2001 should be considered. For beta emitters, double surgical gloves may be helpful in reducing
2002 skin exposures. An intake of airborne material inadvertently released during cutting or
2003 movement of radioactive tissue or organs can be prevented by wearing eye protection and a
2004 face mask.

2005 (218) A proportion of the activity retained will appear in cremated remains and may be
2006 sufficient, particularly in the case of long lived radionuclides, to require controls to be
2007 specified. The main concern is in respect to the scattering of ashes, although contact dose
2008 rates with the container may have to be considered if cremation takes place shortly after
2009 administration.

2010 (219) Crematorium employees may receive external exposure from the radioactive body or
2011 from contamination of the crematorium or internal exposure from inhalation of radioactive
2012 particles while handling the ashes (Wallace and Bush, 1991). Bodies that contain gamma
2013 emitting radionuclides will result in some external exposure to employees of the crematorium.
2014 No precautions are necessary as long as there is minimal time required to handle the body at
2015 the crematorium (a likely assumption). Cremation of non-volatile radionuclides might result
2016 in contamination of the furnace. As the most significant hazard from this contamination is
2017 inhalation of ash particles during cleaning of the furnace, it is appropriate for workers who
2018 clean the furnace to wear dust masks and protective garments.

2019 (220) The most likely hazard to the general population in the vicinity of the crematorium is
2020 the inhalation of radioactive material emitted with the stack gases. Each crematorium should
2021 maintain records of the type and activity in bodies cremated, when known.

2022 **5.5. Comforters and Carers (Medical Exposure), and Members of the**
2023 **Public (Public Exposure)**

2024 (221) *Publication 94* (ICRP, 2004) recommends that young children and infants as well as
2025 visitors not engaged in direct care or comforting should be treated as members of the public
2026 (i.e., be subject to the public dose limit of 1 mSv/y). The registrant or licensee is responsible
2027 for controlling public exposure resulting from a nuclear medicine practice (IAEA, 2011). The
2028 presence of members of the public in or near the nuclear medicine facility shall be considered
2029 when designing the shielding and flow of persons in the facility. The sources of exposure to
2030 the public are primarily the same as for workers. The use of structural shielding and the
2031 control of sources, waste and contamination are thus fundamental to controlling exposure to
2032 the public.

2033 (222) While medical exposures are predominantly delivered to individuals (patients), other
2034 individuals caring for and comforting patients are also exposed to radiation. These
2035 individuals include parents and others, normally family or close friends, who may come close
2036 to patients following administration of radiopharmaceuticals. These exposures are considered
2037 medical exposures (ICRP, 2007a). *Publication 94* (ICRP, 2004) recommends that for
2038 individuals directly involved in comforting and caring (other than young children and infants)
2039 a dose constraint of 5 mSv per episode (i.e., for the duration of a given release from hospital
2040 after therapy) is reasonable. The constraint needs to be used flexibly. For example higher
2041 doses may well be appropriate for parents of very sick children.

2042 **5.5.1. Release of the patient**

2043 (223) A patient who has undergone a therapeutic nuclear medicine procedure is a source of
2044 radiation that can lead to the exposure of other persons who come into the proximity of the
2045 patient. External irradiation of the persons close to the patient is related to the radionuclide
2046 used, its emissions, half-life and biokinetics, and can be important for some radionuclides.
2047 Excretion and vomitus result in the possibility of contamination of the patient's environment
2048 and other persons.

2049 (224) If a non-occupationally exposed person is knowingly and voluntarily providing care,
2050 comfort and support to the patient, then their exposure is considered part of medical exposure,
2051 and they are subject to dose constraints (ICRP, 2007b). If the person is simply a member of
2052 the public, including persons whose work in the nuclear medicine facility does not involve
2053 working with radiation, then their exposure is part of public exposure.

2054 (225) Patients do not need to be hospitalised automatically after all radionuclide therapies.
2055 Relevant national dose limits must be met and the principle of optimisation of protection
2056 must be applied, including the use of relevant dose constraints. The decision to hospitalise or
2057 to release a patient should be determined on an individual basis considering factors such as
2058 radiation level of the patient measured by dose rate monitoring, the residual activity in the
2059 patient, the patient's wishes, family considerations (particularly the presence of children),
2060 environmental factors, and existing guidance and regulations. Hospitalisation will reduce
2061 exposure to the public and relatives, but will increase exposure to hospital staff.
2062 Hospitalisation often involves a significant psychological burden as well as monetary and
2063 other costs that should be analysed and justified. ICRP (2004) has given detailed
2064 recommendations related to release of patients after therapy with unsealed radionuclides in its
2065 *Publication 94* (ICRP, 2004).

2066 (226) Current recommendations regarding release of patients after therapy with unsealed
2067 radionuclides vary widely around the world. However, the decision to release a patient is
2068 based on the assumption that the risk can be controlled when the patient returns to their home.
2069 This is generally achieved by combining an appropriate release criterion with well-tailored
2070 instructions and information for the patient that will allow them to deal effectively with the
2071 potential risks.

2072 (227) When appropriate, the patient or legal guardian shall be provided with written and
2073 verbal instructions with a view to the restriction of doses to persons in contact with the
2074 patient as far as reasonably achievable, and information on the risks of ionising radiation. It is
2075 important to develop effective communication methods. Specific instructions should include:
2076 minimisation of the spread of contamination, minimisation of exposure to family members,
2077 cessation of breast-feeding, and delaying conception after therapy. The amount of time that
2078 each precaution should be implemented should be determined based upon an estimate of the
2079 activity in the patient prior to discharge and an assessment of the dose likely to be received
2080 by carers and comforters or members of the public under various precaution formulations as
2081 compared to the appropriate dose constraints. Procedures for advising carers and comforters
2082 should be in place, developed in consultation with the RPO. Registrants and licensees should
2083 ensure that carers and comforters of patients during the course of treatment with
2084 radionuclides receive sufficient written instructions on relevant radiation protection
2085 precautions (e.g. time and proximity to the patient). Example methodologies for evaluating
2086 precaution time requirements have been published (Zanzonico et al., 2000; NCRP, 2006;
2087 IAEA, 2009; Sisson et al., 2011).

2088 (228) Travel following therapy should be within certain restrictions and patients should
2089 carry relevant documentation in case of a medical emergency. If travelling, radiation
2090 detectors used for security purposes, for example in airports, are sufficiently sensitive to
2091 detect low levels of radiation.

2092 **5.5.2. Visitors to patients**

2093 (229) Arrangements should be made to control access of visitors (with special emphasis on
2094 controlling access of pregnant visitors and children) to patients undergoing
2095 radiopharmaceutical therapy and to provide adequate information and instruction to these
2096 persons before they enter the patient's room, so as to ensure appropriate protection. Licensees
2097 should also take measures for restricting public exposure to contamination in areas accessible
2098 to the public.

2099 **5.5.3. Travel**

2100 (230) Optimally, when there is no physical or other impairment, the patient should drive
2101 alone in a private car. If the patient must ride or drive with another person, then time and
2102 distance constraints apply. Use of a larger vehicle, such as a van, would permit further
2103 separation and consequently a reduction in exposure to others. The ICRP has previously
2104 evaluated the potential doses to others during patient travel and have published
2105 recommendations that allow use of public transportation by some patients treated by nuclear
2106 medicine therapy (ICRP, 2004 – see Table 10.7). Radionuclide characteristics and activity
2107 administered should be considered. For example, for patients treated for hyperthyroidism, the
2108 patient may use public transportation for up to 0.5 h if treated with 800 MBq or up to 3.5 h if
2109 treated with 200 MBq (ICRP, 2004).

2110 (231) Patients travelling after radioiodine therapy rarely present a hazard to other
2111 passengers if travel times are limited to a few hours. Travel for 1–2 h immediately post-
2112 treatment in a private automobile large enough for the patient to maintain a distance of 1 m or
2113 greater from the other vehicle occupant(s) is generally permissible. A case-by-case analysis is
2114 necessary to determine the actual travel restrictions for each patient, especially for longer
2115 trips and for travel by public transport. A stay in a hotel or motel is not recommended after
2116 treatment with nuclear medicine therapy without specific environmental assessments and
2117 dose-rate evaluations. Exposure of those immediately involved with the patient and the
2118 general population can occur through environmental pathways including sewerage,
2119 discharges to water, incinerated sludge or cremation of bodies. From the point of view of the
2120 individual doses involved, this is of relatively minor significance (IAEA, 2009).

2121 (232) Current international security measures, such as those in place at airports and border
2122 crossing points, can include extremely sensitive radiation detectors. It is quite possible that
2123 patients treated with gamma-emitting radionuclides could trigger these alarms, particularly in
2124 the period immediately following discharge. With current technology, it is possible to detect
2125 ^{131}I activity as little as 0.01 MBq at 2 to 3 m (Dauer et al., 2007a). It is possible that patients
2126 treated with radionuclides could trigger alarms for 95 days or longer (Dauer et al., 2007b,
2127 2007c). Triggering of an alarm does not mean that a patient is emitting dangerous levels of
2128 radiation, as the detectors are designed to detect levels of radioactivity far below those of
2129 concern to human health. The security authorities are well aware of this possibility, and if a
2130 patient is likely to travel soon after discharge, the hospital or the patient's doctor should
2131 provide a written statement of the therapy and radionuclide used for the patient to carry.
2132 Personnel operating such detectors should be specifically trained to identify and deal with
2133 nuclear medicine patients. Records of the specific details of therapy with unsealed
2134 radionuclides should be maintained at the hospital and given to the patient along with written
2135 precautionary instructions (ICRP, 2008).

2136 (233) If travel is planned within 4 months of receiving radiopharmaceutical therapy,
2137 particularly across international borders or via airports, tunnels, and/or over bridges or
2138 whenever inspection is likely, a form or card should be provided to the patient (Sisson et al.,
2139 2011). The form should specify the date of treatment, the radionuclide activity administered,
2140 the treating facility, and the name and telephone number of a contact individual
2141 knowledgeable about the case.

2142 **5.5.4. Radioactive waste**

2143 (234) Licensees are responsible for ensuring that the optimisation process for measures to
2144 control the discharge of radioactive substances from a source to the environment is subject to
2145 dose constraints established or approved by the regulatory body (IAEA, 2000, 2004, 2005a).
2146 This is particularly relevant for facilities where exhaust systems are required for radioiodine
2147 storage and handling. The need for containment and/or ventilation for accumulated or stored
2148 ^{131}I waste should be evaluated where appropriate. While for diagnostic patients there is no
2149 need for collection of excreta and ordinary toilets can be used, for therapy patients, there are
2150 very different policies in different countries, but, in principle, the clearance criteria should
2151 follow a dilution and decay methodology. Much of the activity initially administered is
2152 eventually discharged to sewers. Storing a patient's urine after therapy appears to have
2153 minimal benefit as radionuclides released into modern sewage systems are likely to result in
2154 doses to sewer workers and the public that are well below public dose limits (ICRP, 2004).
2155 However, local restrictions regarding the discharge of activity may apply. Once a patient has

2156 been released from hospital, the excreted radioactivity levels are low enough to be discharged
2157 through the toilet in their home without exceeding public dose limits.
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6. SUMMARY OF RECOMMENDATIONS

2160 (235) Increasing use of radiopharmaceuticals for therapy is of benefit for the patient. The
2161 goal of radiation therapy, including therapy with radiopharmaceuticals, is to optimise the
2162 relationship between the probability of tumour control and the probability of normal tissue
2163 complications.

2164 (236) In radiopharmaceutical therapy, the absorbed dose in an organ or tissue is governed
2165 by the radiopharmaceutical uptake and retention in the organ itself and surrounding organs,
2166 combined with the radionuclide physical half-life. Biokinetic data are collected using
2167 techniques that vary in complexity and chosen with regard to the accuracy required for the
2168 particular task.

2169 (237) Individual dose estimates must be performed for each patient. In principle, a fully
2170 personalised approach based on patient-specific measurements can ensure the administration
2171 of appropriate activity for treatment with minimal effects in surrounding normal tissue,
2172 thereby minimising the radiation doses delivered to staff, family and comforters and carers
2173 and will further minimise the long-term risks.

2174 (238) Special consideration should be given to pregnant women exposed to ionising
2175 radiation. Pregnancy is a strong contraindication to radiopharmaceutical therapy, unless the
2176 therapy is life-saving. Female patients should be advised that breastfeeding is contraindicated
2177 after therapeutic administration of radionuclides. Breastfeeding should be discontinued in
2178 radiopharmaceutical therapy patients.

2179 (239) In addition to the patients treated with radiopharmaceutical therapy, the people at
2180 risk of exposure include hospital staff, members of the patient's family, including children,
2181 and carers, neighbours, and the general public. These risks can be effectively managed and
2182 mitigated with well-trained staff, appropriate facilities, and the use of patient-specific
2183 radiation safety precaution instructions.

2184 (240) Optimisation of staff exposures include consideration of equipment design, proper
2185 shielding and handling of sources as well as personal protective equipment and tools as well
2186 as education and training resulting in awareness and engagement in radiation protection.
2187 Individual monitoring of the whole body and extremities must be considered during the
2188 management of radiopharmaceutical therapy patients and in the preparation and
2189 administration of radiopharmaceuticals.

2190 (241) Medical practitioners should provide all necessary medical care consistent with
2191 patient safety and appropriate medical management. Radiological protection considerations
2192 should not prevent or delay life-saving operations in the event that surgery is required. Staff
2193 should be informed when a patient may pose a radioactive hazard, and advice and training
2194 should be provided prior to administrations.

2195 (242) The decision to hospitalise or release a patient after therapy should be made on an
2196 individual basis considering factors such as the residual activity in the patient, the patient's
2197 wishes, family considerations (particularly the presence of children), environmental factors,
2198 and existing guidance and regulations. Specific radiation protection precautions should be
2199 provided to patients and carers.

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